

Multigroup Data Libraries for SCALE Applications

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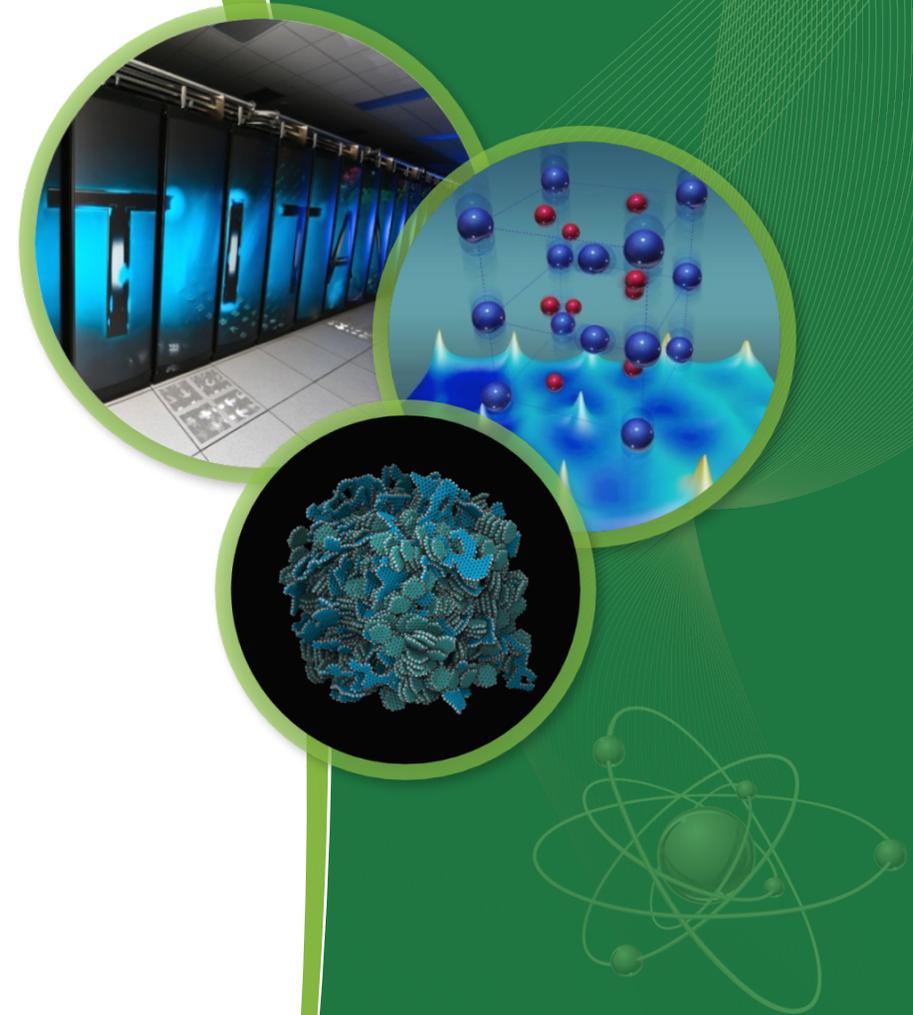
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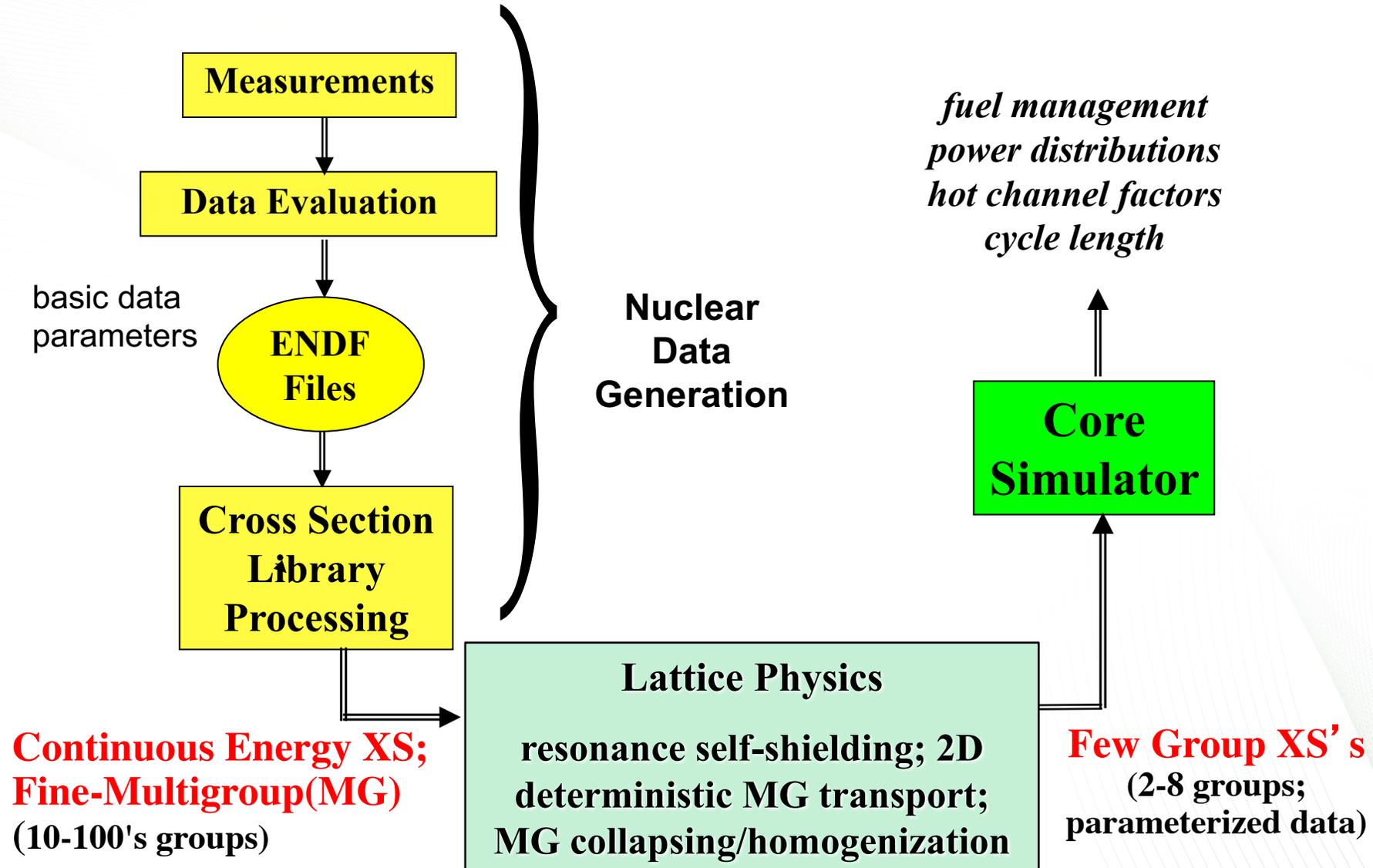
SCALE Users' Group Workshop

Oak Ridge, TN.

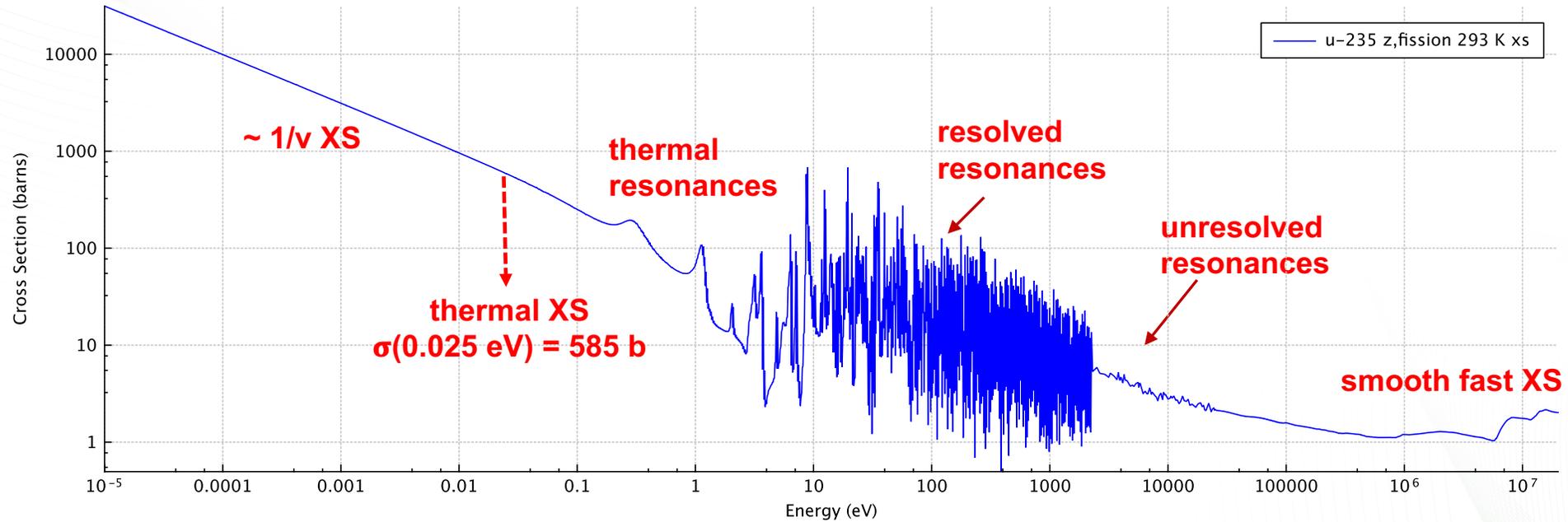
Sept. 27, 2017



Cross-Section Generation

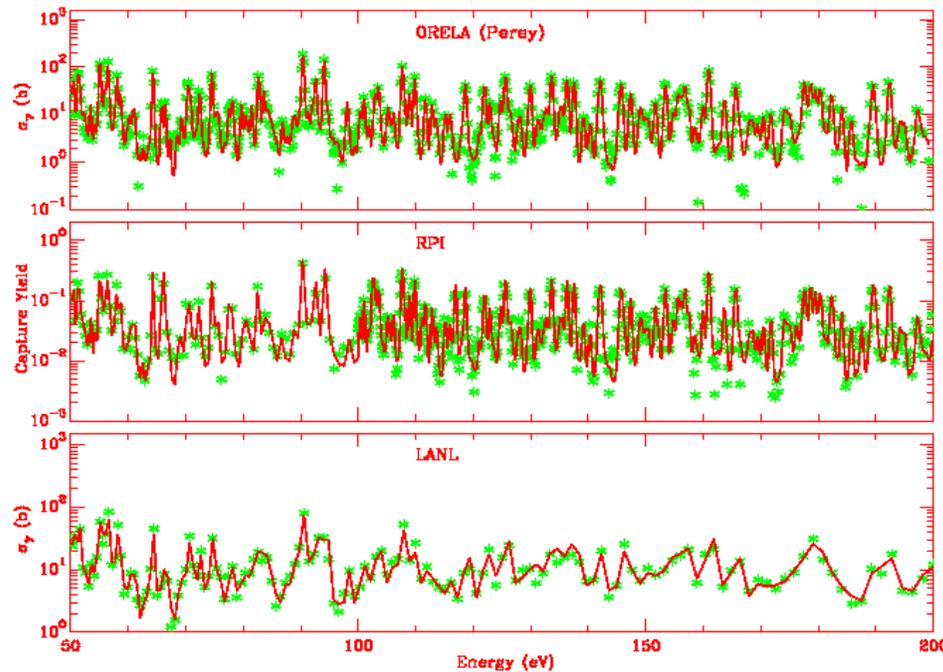


U235 Fission Cross Section

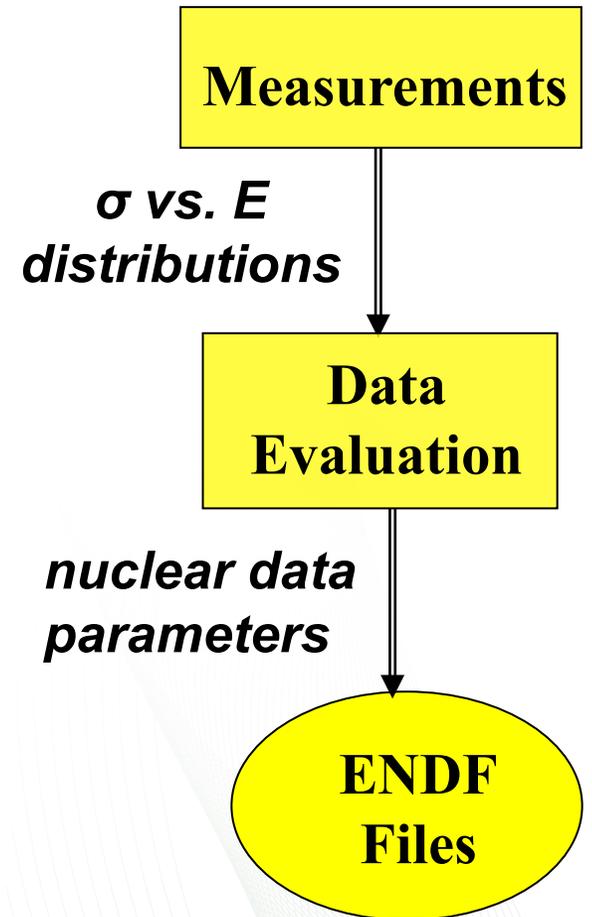


Evaluated Nuclear Data

- Experimental measurements provide interaction probabilities $[\sigma(E)]$ vs. neutron energies
- Theoretical expressions $\sigma(E)$ can be derived from quantum mechanics
- Regression of parameters in nuclear physics models performed to best fit measured values
- Evaluated parameters and $\sigma(E)$ stored in ENDF/B files



**ORNL Evaluation of U-235
Capture Data 50 – 200 (eV)**

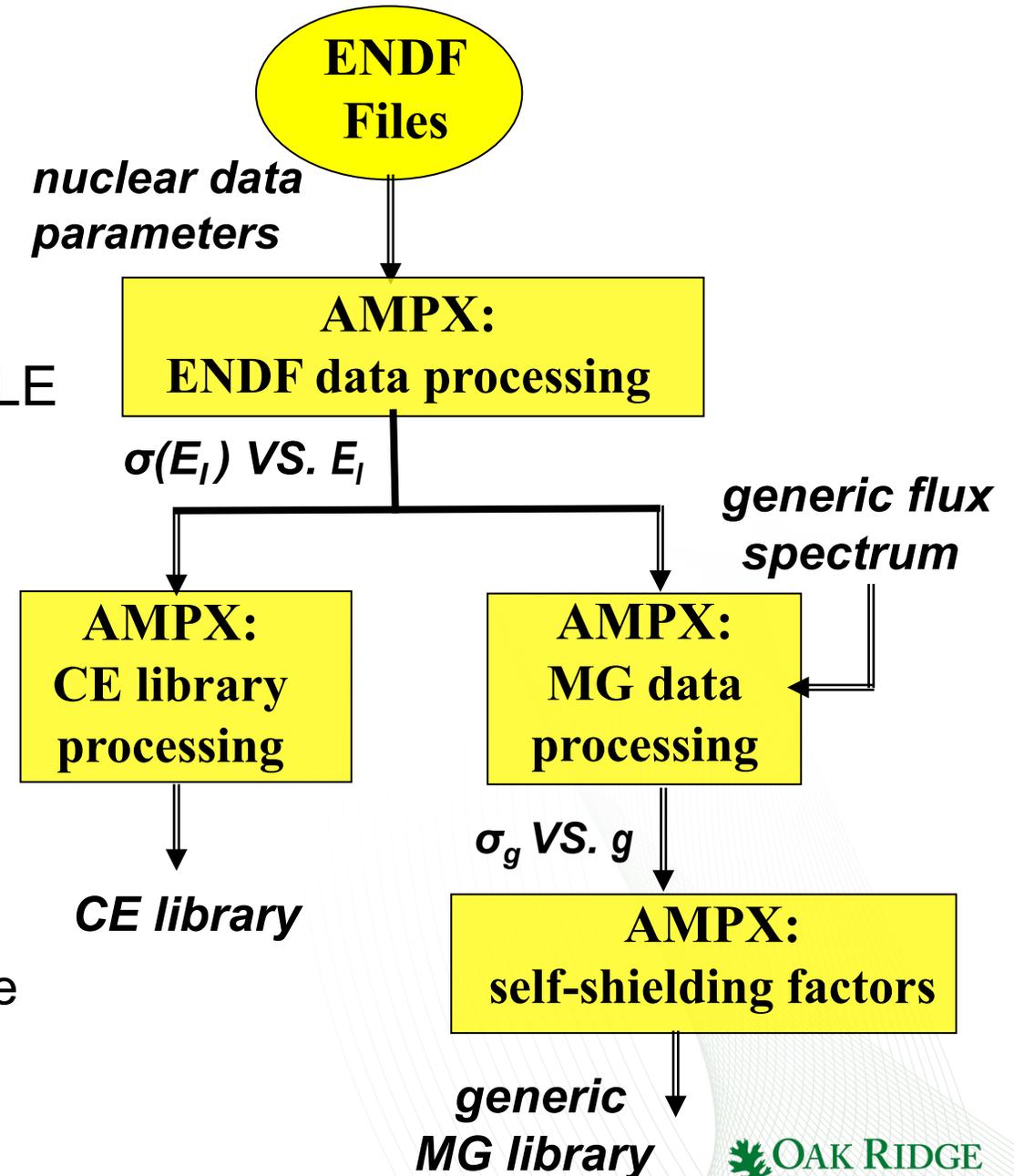


ENDF/B-VII.1 Contains Evaluated Nuclear Data for 423 Nuclides

File	Description	File	Description
1	General Information	10	Cross Sections for the Production of Radioactive Nuclides
2	Resonance Parameters	11	General Comments of Photon Production
3	Neutron Cross Sections	12	Photon Production and Multiplicities and Transition Probability Arrays
4	Angular Dist. of Secondary Particles	13	Photon Production Cross Sections
5	Energy Dist. of Secondary Particles	14	Photon Angular Distributions
6	Coupled Energy-Angle Dist. of Secondary Particles	15	Continuous Photon Energy Spectra
7	S(α , β) Scattering Law Data	23	Photon Interaction Cross Sections
8	Radioactive Decay and Fission Product Data	27	Atomic Form Factors or Scattering Functions
9	Multiplicities for Production of Radioactive Nuclides	30 – 40	Data Covariance Files

Nuclear Data Processing For SCALE

- **AMPX** code system reads ENDF/B files and processes data to *continuous energy* (CE) or *multigroup* (MG) libraries distributed with SCALE
- **CE data library processing**
 - Converts ENDF data to linearized “point data” $\sigma(E_i)$ VS. E_i and energy-angle distributions
 - Convert point data to probability distributions for Monte Carlo
- **MG data library processing**
 - Determine MG energy structure
 - Average CE data over specified group structure using “generic” energy spectrum
 - Compute resonance self-shielding data



SCALE MG Libraries Contain Generic Cross Sections and Self-Shielding Data

$$\sigma_g \equiv \frac{\int_g \phi(E) \sigma(E) dE}{\int_g \phi(E) dE}$$

- $\sigma(E)$ = energy-dependent CE data processed from ENDF/B
- $\Phi(E)$ = weight-function representing the energy spectrum

- *Problem-Independent MG Data (Generic, Infinitely Dilute)*

$\Phi \rightarrow \Phi_\infty(E)$, “generic” function defined as the asymptotic flux in the limit of negligible resonance absorption. Does not depend on specific composition or geometry.

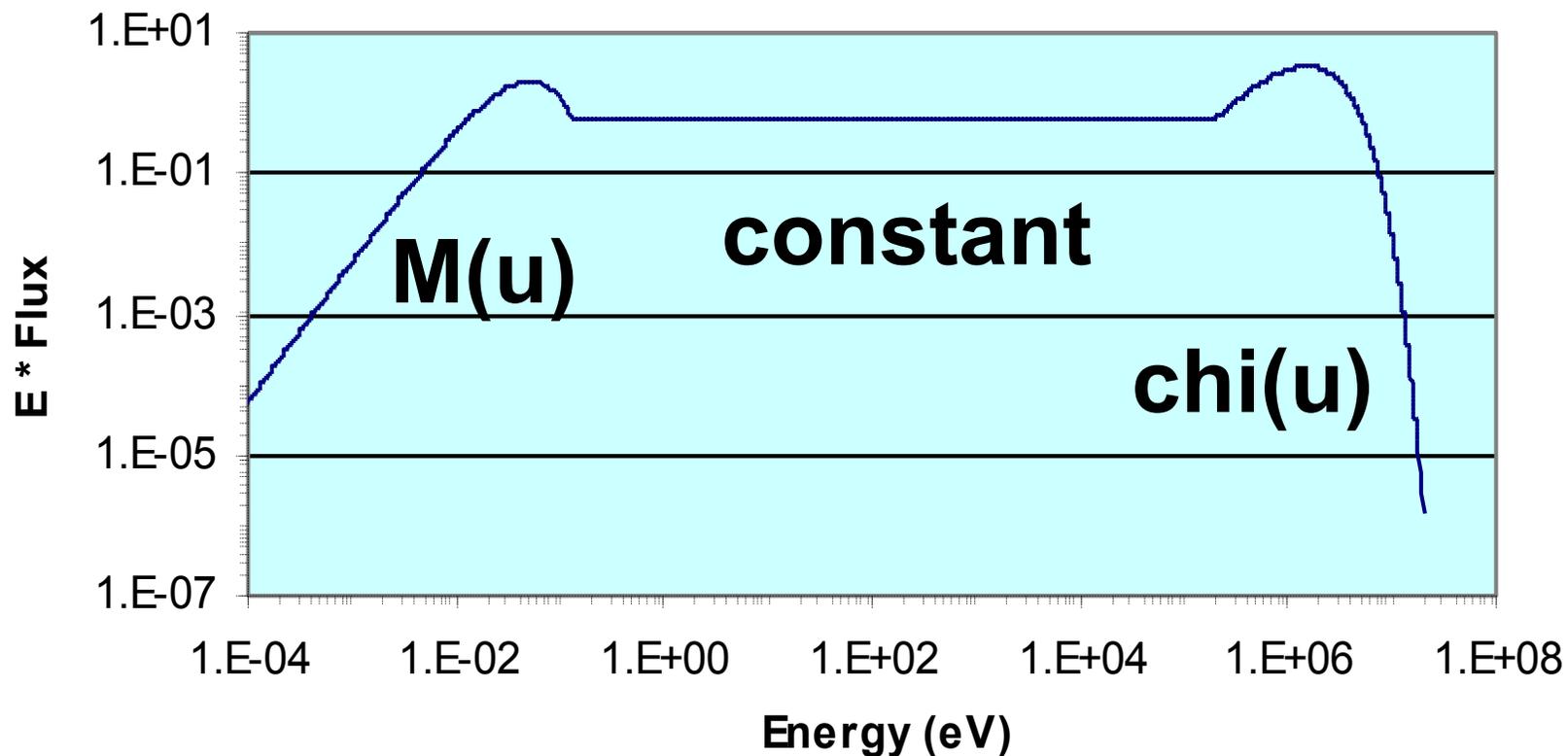
- *Problem-Dependent MG Data (Self-Shielded)*

$\Phi \rightarrow \Phi_{sys}(E)$, the flux spectrum for the actual system of interest, accounting for resonance absorption effects. Depends on the actual absorber/moderator concentrations and geometry.

resonance absorption corrections are made to the generic library when used for a particular application.

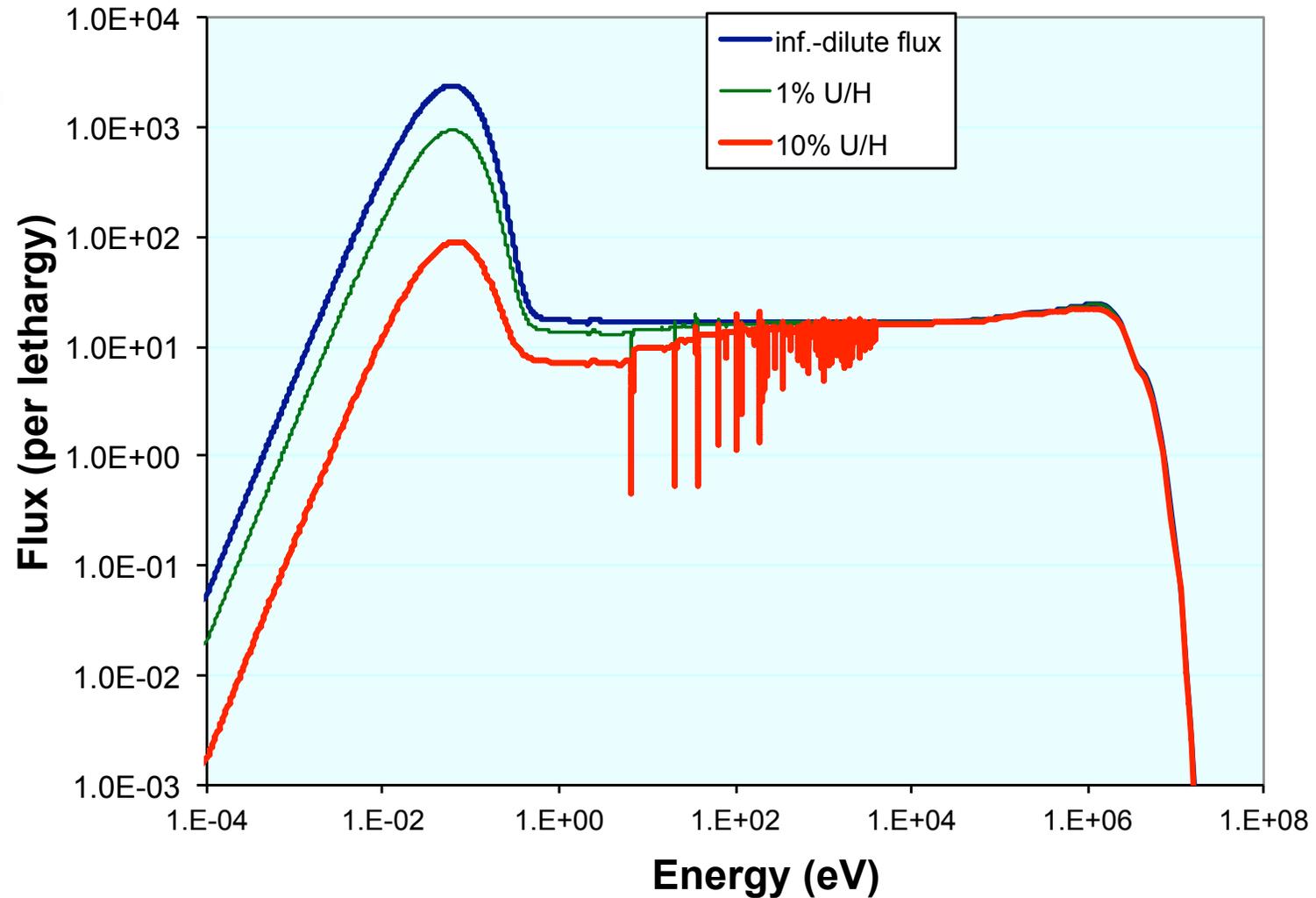
“Generic” Neutron Spectrum: Fission Source in a Moderator *(Infinitely Dilute Resonance Absorbers)*

Asymptotic Flux per lethargy



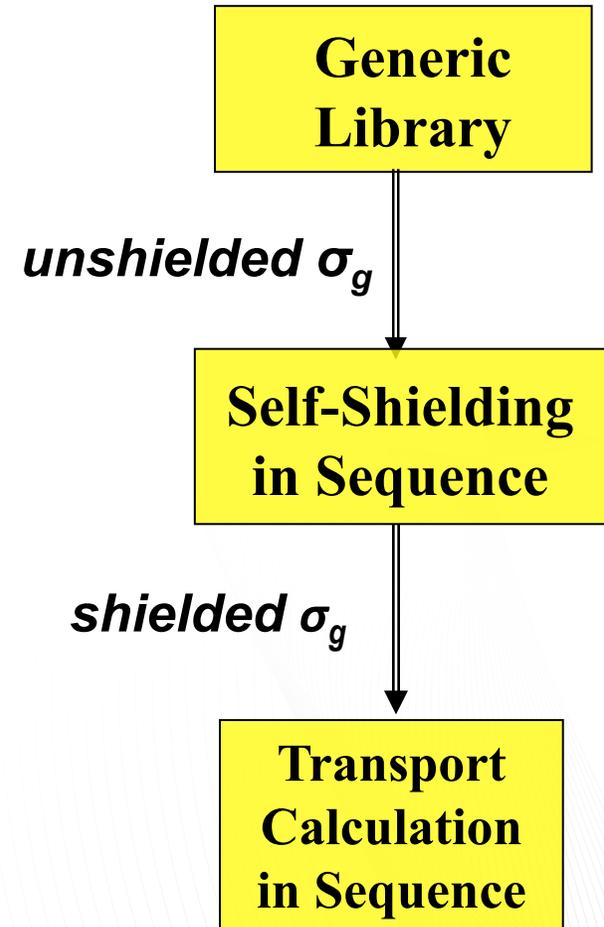
Impact of Resonance Absorption on Neutron Spectrum

Flux spectrum in infinite medium of U-238 and H



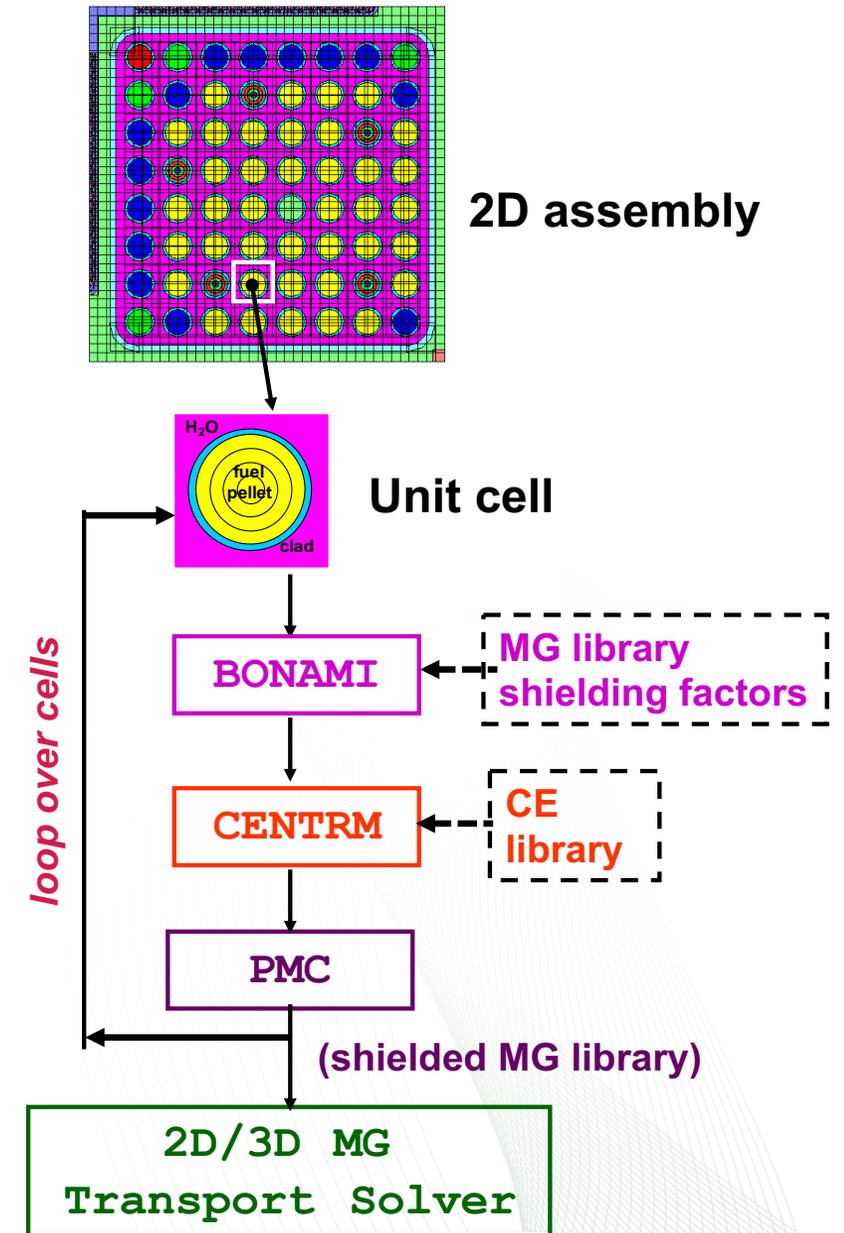
Conversion of Generic Library Cross Sections to Problem-Specific Self-Shielded Data

- Self-shielded data for transport calculations are computed during execution of SCALE sequences
- SCALE has two different methods for self-shielding
 - XSProc procedure default for CSAS and TRITON sequences
 - Bondarenko method default for Mavric and Polaris

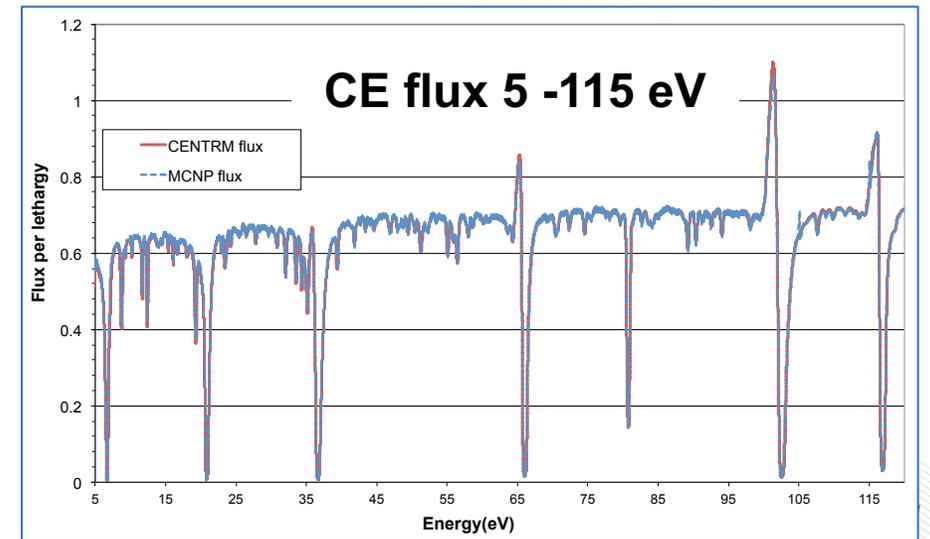
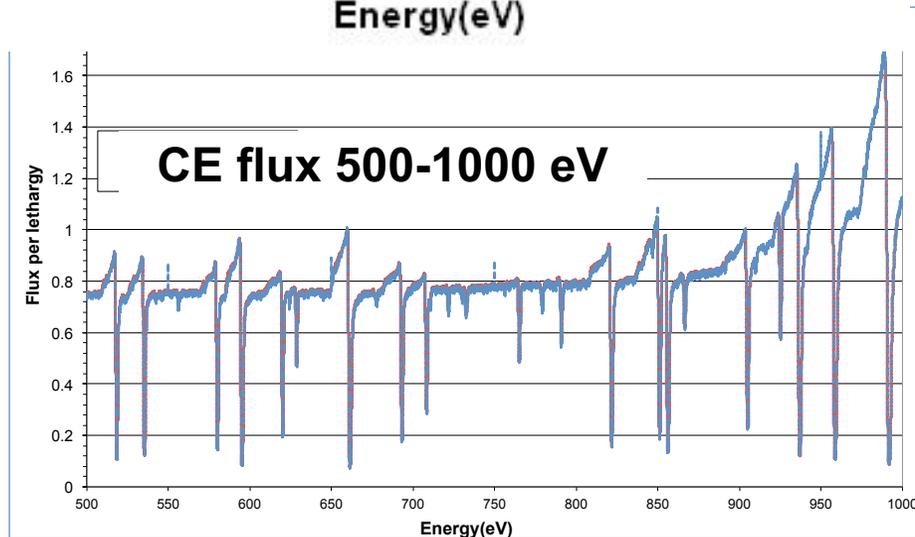
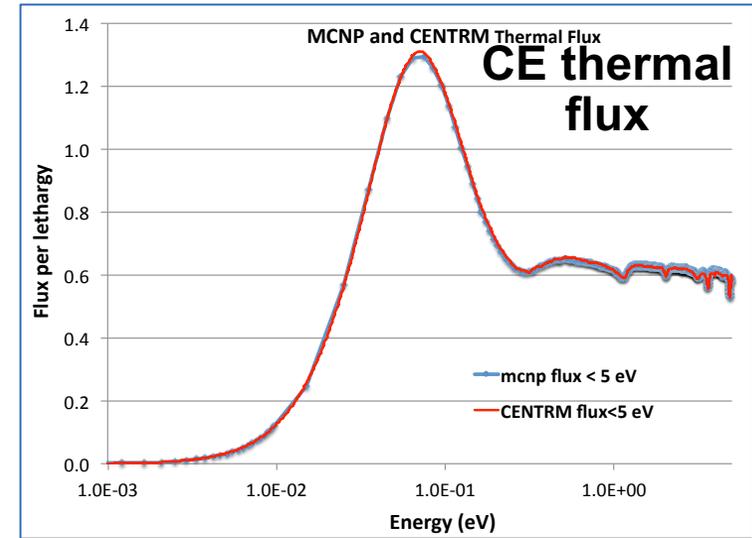
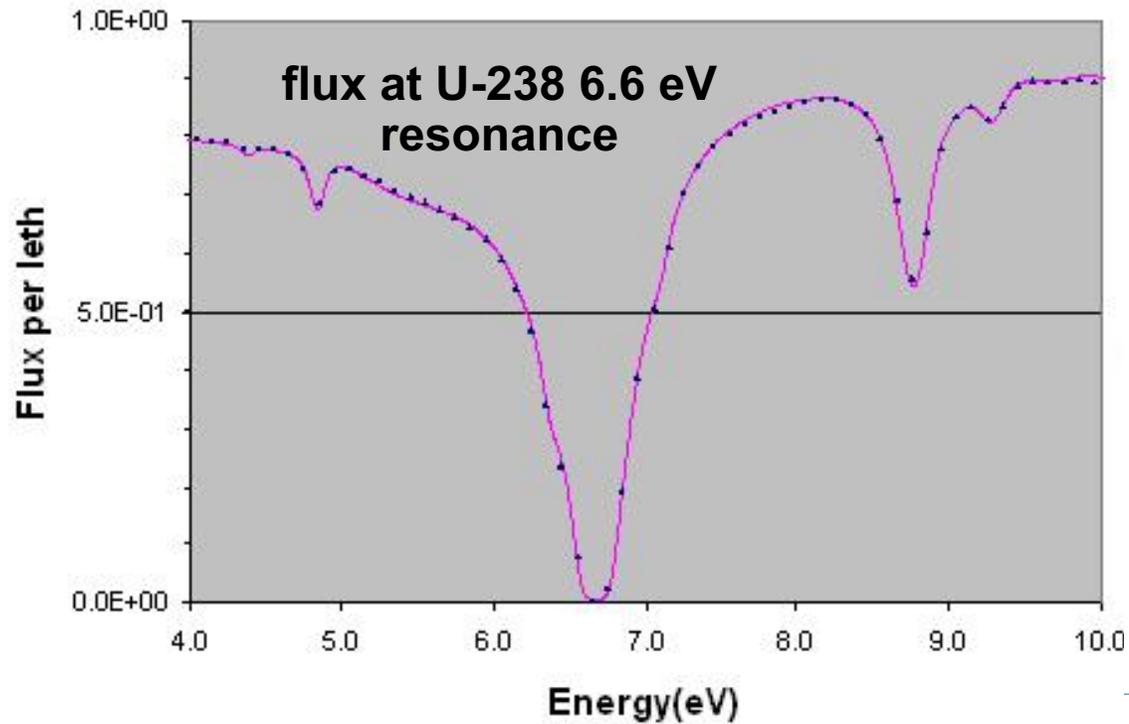


SCALE Resonance Self-Shielding with XSProc Module

- XSProc is default method for CSAS (criticality), TRITON (general lattice physics)
 - BONAMI
 - Uses Bondarenko method with pre-computed self-shielded data stored in library
 - Interpolates library data to obtain shielded data in unresolved and fast range (and optionally in resolved range)
 - CENTRM/PMC
 - Generates "on-the-fly" shielded data using problem-specific spectra from CE deterministic calculations in resolved range

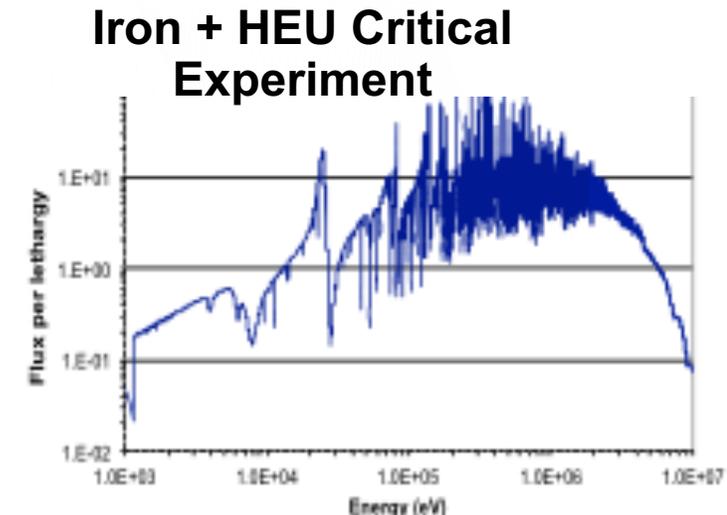
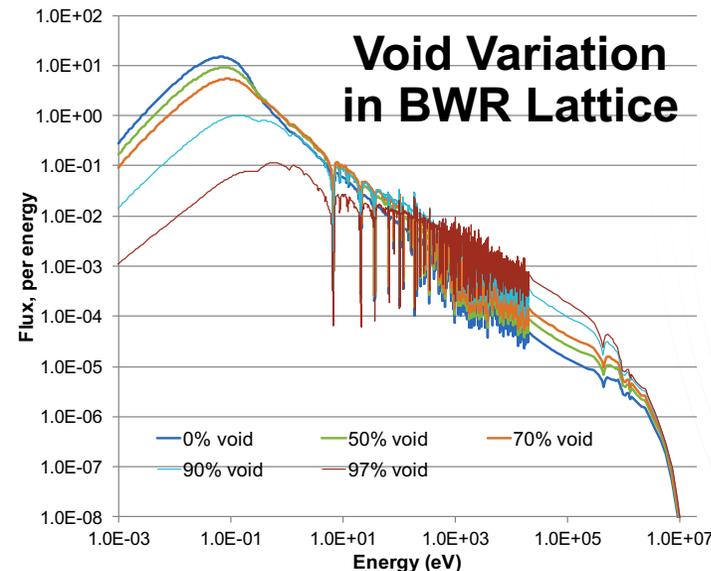
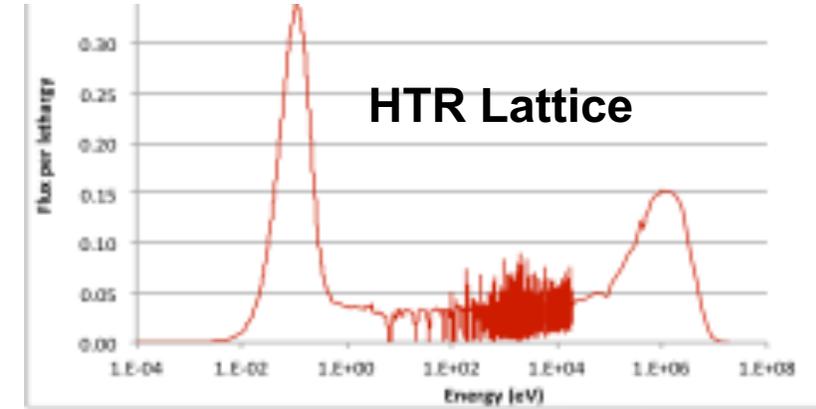


CENTRM and MCNP Flux Spectra for PWR Lattice



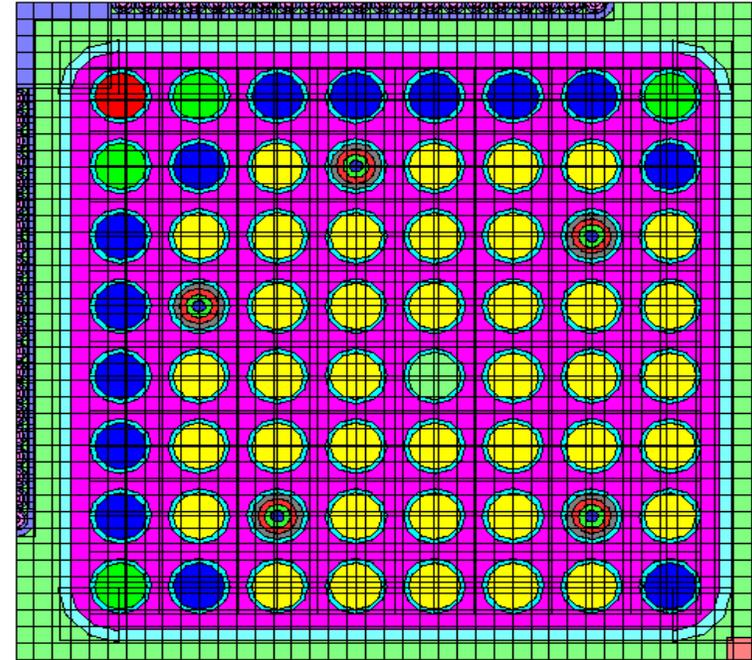
CENTRM/PMC Provides Capability to Address Resonance Effects Using First Principles

- Performs deterministic MoC or SN transport with CE library point data (70,000-100,00 energy points)
- Provides problem-specific CE weighting function to process MG data on-the-fly
- Explicit resonance overlap/interference
- Inherent space-dependent self-shielding
- Self-shielded elastic removal
- Includes impact of thermal scatter on self-shielding



Bondarenko Methods Use Pre-computed Shielded Data Spanning Range of Self-Shielding Conditions

- Bondarenko is more approximate but faster than on-the-fly CENTRM/PMC
- Factors impacting self-shielding in lattices
 - Concentration of resonance nuclide (N_r)
 - Concentration of admixed moderators (Σ_m)
 - Composition of external moderators (Σ_{M^*})
 - Dimension/shape of fuel pins (e.g., *fuel radius* = R_f)
 - Arrangement of fuel pins (e.g., *pitch* = P)
 - Temperatures = T

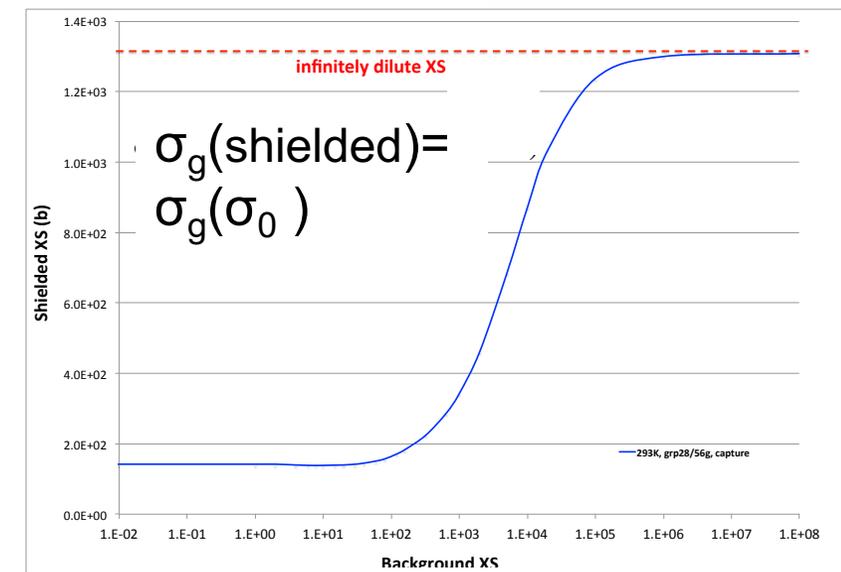
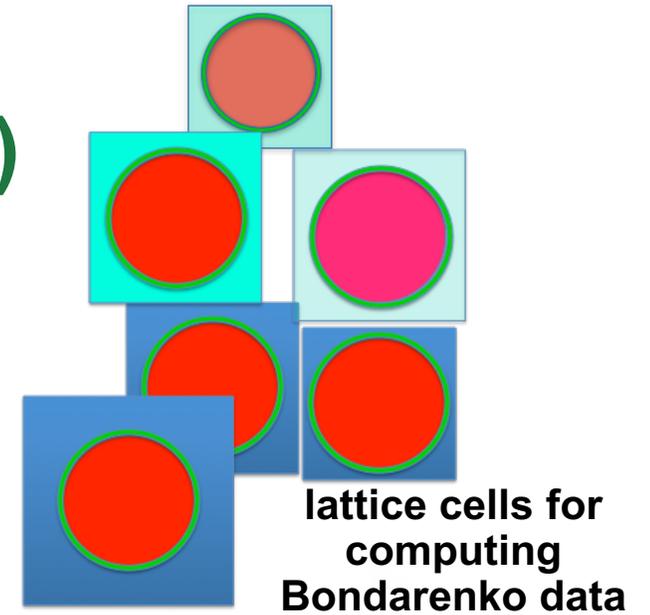


$$\sigma_{g(\text{shielded})} = \sigma_g(N_r, \Sigma_m, \Sigma_{M^*}, R_f, P, T_F)$$

- “Equivalence theory” background cross section: $\sigma_0 \rightarrow (N_r, \Sigma_m, \Sigma_{M^*}, R_f, P, T_F)$

AMPX Calculation of Bondarenko Shielding Data with CENTRM/PMC (IRFfactor module)

- CENTRM/PMC calculate range of shielded cross sections for library by varying self-shielding conditions
- Heterogeneous lattice models used for U, Pu, Gd, Zr
 - Vary lattice parameters ($N_r, \Sigma_m, \Sigma_{M^*}, R_f, P, T_F$) $\rightarrow \sigma_0$
 - Compute self-shielded data with CENTRM/PM for all σ_0
- Bondarenko data for other nuclides computed from homogeneous media models
 - Vary absorber/moderator number densities (N_r, Σ_m) $\rightarrow \sigma_0$
- Problem-dependent, self-shielded cross sections for lattice physics:
 - determine value of σ_0 ($N_r, \Sigma_m, \Sigma_{M^*}, R_f, P$) for system
 - interpolate tabulated library values to obtain $\sigma_g(\sigma_0)$

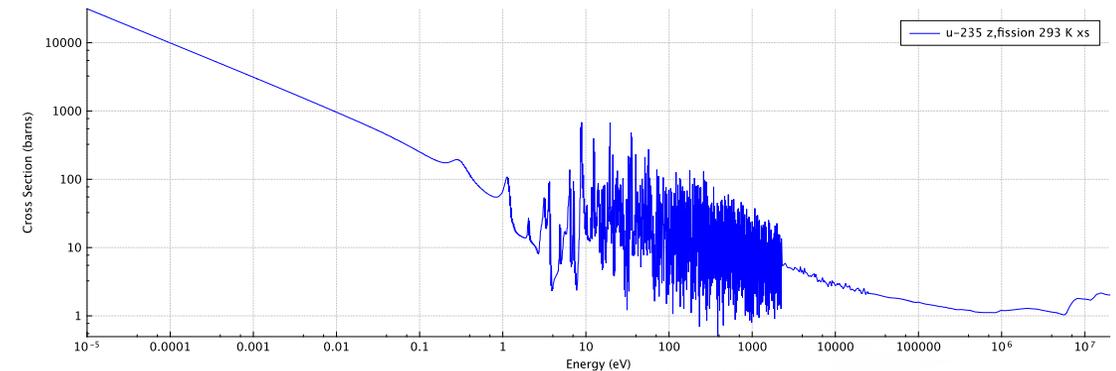


U-238 capture vs. σ_0
(group 121/252)

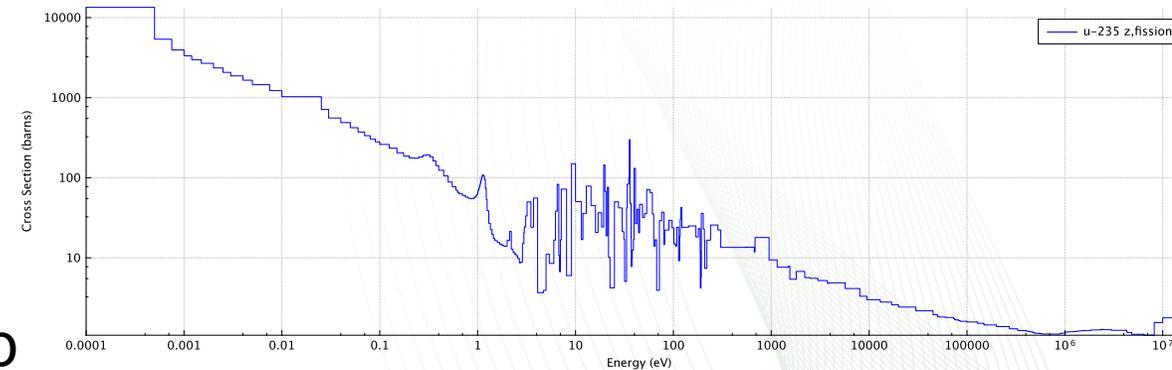
ENDF/B-VII.1 and VII.0 Nuclear Data Libraries in SCALE

- CE neutron/gamma library for Monte Carlo
 - general applications
- 252 fine-group
 - general reactor physics
 - criticality Safety
- 56 broad-group
 - LWR reactor physics
- 200 neutron/47 gamma coupled fine-group
 - shielding
 - fast-reactor analysis
- 28 neutron/19 gamma coupled broad-group
 - shielding

U235 Fission XS



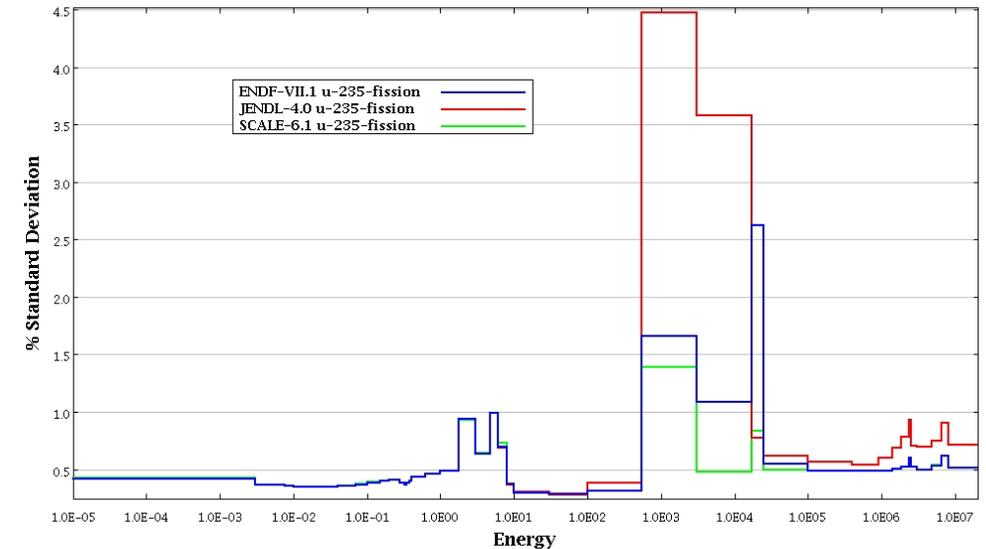
CE data



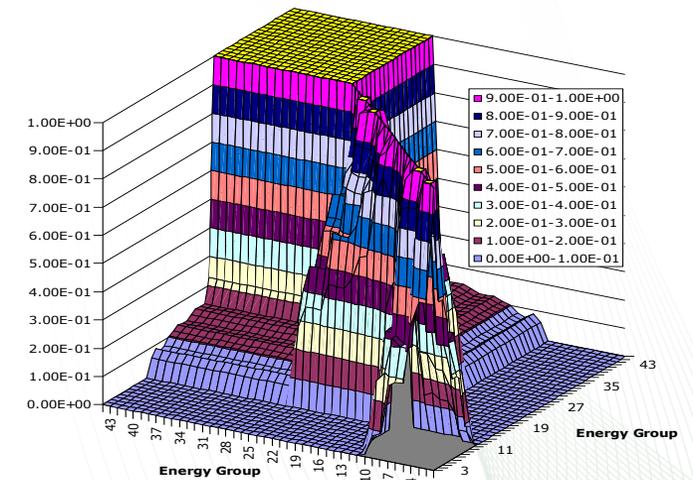
252 MG data

SCALE MG Covariance Libraries

- Evaluated nuclear data have uncertainties from uncertainties in measurements and evaluation techniques
- Uncertainties are described by covariance data
 - Standard deviations
 - Correlation matrices
- SCALE includes 252 and 56-group covariance libraries generated from several sources
 - Covariance data for 187 nuclides from ENDF/B VII.1
 - Covariances for ~215 nuclides from other sources
- SCALE modules TSUNAMI and Sampler compute uncertainty in results using covariance libraries



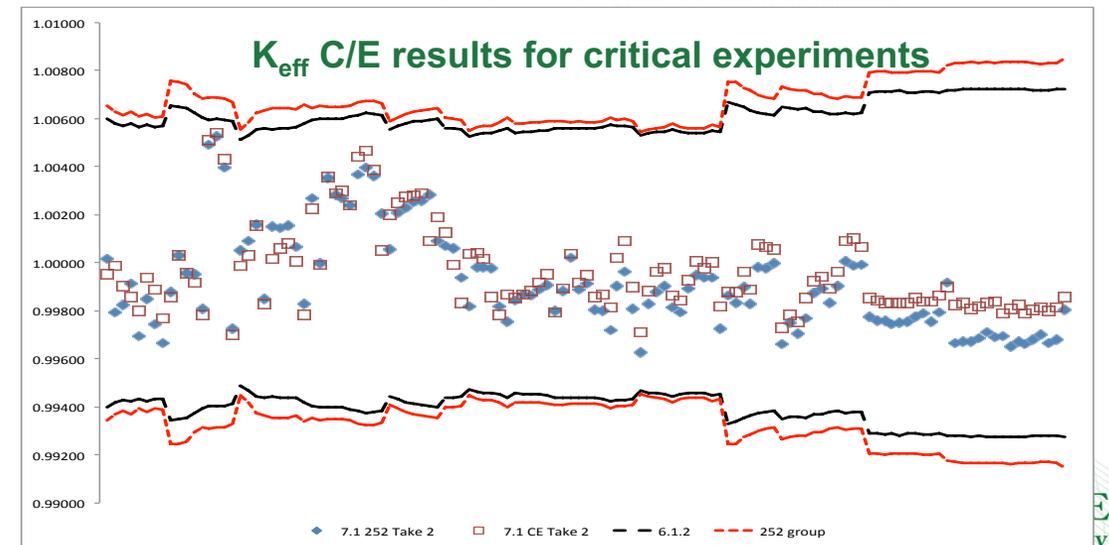
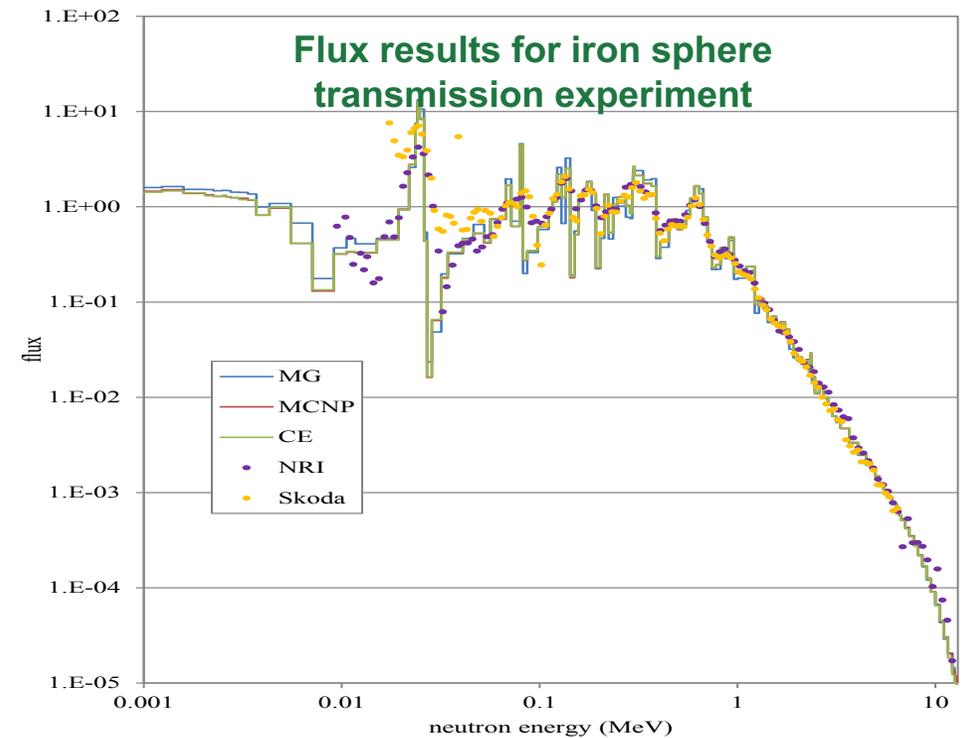
standard deviation In U-235 fission



U-235 fission correlations

Validation and Nuclear Data Testing

- >5000 infinite medium k_{inf} tests
 - Every nuclide, differing moderation
- >7000 fixed-source transmission tests for neutron/gamma spectral data
 - Every nuclide/element at multiple energies
- 400 criticality benchmarks
 - HEU and LEU solutions
 - Pu solutions
 - LEU and Mox lattices
 - U and Pu metal fuels
- Shielding benchmarks
 - Fission and 14 MeV sources



SUMMARY

- AMPX code system processes ENDF/B-VII into SCALE CE and MG nuclear data
- SCALE-6.2 includes CE libraries for Monte Carlo codes Keno and Monaco, and for CENTRM/PMC
- “Generic” fine and broad MG libraries are available for lattice physics, criticality safety, and shielding applications
- Generic MG data is converted into problem-specific data using self-shielding modules in SCALE
 - On-the-fly CENTRM/PMC is default for Triton, CSAS sequences
 - Bondarenko method is default for Mavric and Polaris sequences
- MG libraries include Bondarenko self-shielding data pre-computed with CENTRM/PMC
- Extensive testing of SCALE CE and MG libraries was performed for thousands of verification tests and for ~ 400 critical benchmark experiments