

# Research reactor source terms with ORIGEN for safeguards applications - Tutorial

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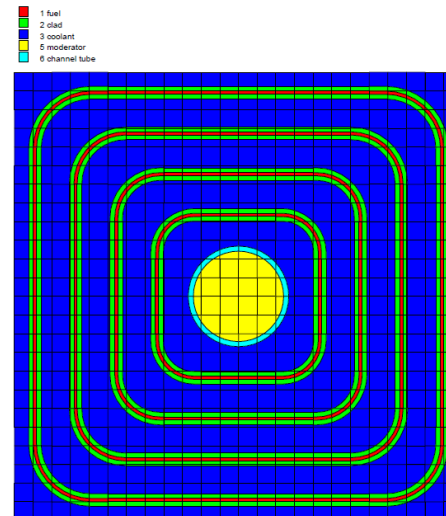
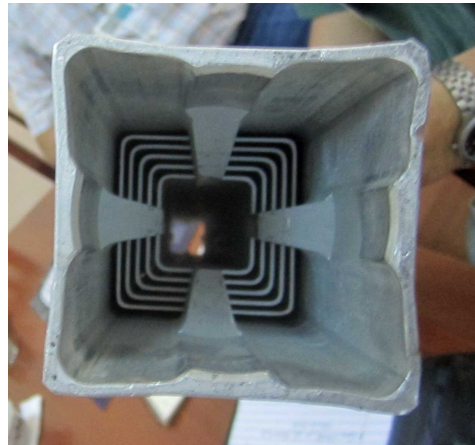
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# Problem Description

- Generate neutron and gamma spectra for IRT research reactor spent fuel using ORIGEN.
- Calculate total U and total Pu inventory in spent fuel at 5 years after discharge.
- Perform calculations for LEU fuel (IRT-4M 6-tube 19.75%  $^{235}\text{U}$ ) and HEU fuel (IRT-3M 6-tube 90%  $^{235}\text{U}$ ) and compare LEU and HEU results.

# Learning Goals

- How to use existing or user-generated ORIGEN research reactor libraries with ORIGEN under Fulcrum or Origami GUI.
- How to calculate isotopic inventories, neutron and gamma sources using an available ORIGEN research reactor library.
- Estimate the difference in radiation sources between LEU and HEU spent fuel.



# ORIGEN Reactor Libraries in SCALE

- Fuel/reactor specific ORIGEN libraries reside in the SCALE subdirectory `SCALE-6.2\data\arplibs\`
- The file `arpdata.txt` contains information on the libraries
  - Fuel type (name of the reactor library)
  - Number of values for each variable parameter
  - Parameter values
  - Burnup values for each library position
  - Filenames for parameter-dependent libraries
- The file `arpdata.txt` is located in `SCALE-6.2\data\`

# Example - Information in arpdata.txt for IRT-4M 6-tubes

**!irt4m6tube**

**1 1 32**

**19.7500**

**0.9890**

**'irt4m6t\_e19.f33'**

**0.00 900.00 3600.00 8100.00 13500.00**  
**18900.00 24300.00 29700.00 35100.00 40500.00**  
**45900.00 51300.00 56700.00 62100.00 67500.00**  
**72900.00 78300.00 83700.00 89100.00 94500.00**  
**99900.00 105300.00 110700.00 116100.00 121500.00**  
**126900.00 132300.00 137700.00 143100.00 148500.00**  
**153900.00 159300.00**

**library name**

**1 enrichments, 1 moderator density, 32 burnups**

**enrichment (wt% U-235)**

**moderator density (g/cm3)**

**names of library files**

**burnups (in MWd/MTU)**

# How to use arpdata.txt

- ORIGEN (and Origami) will look for reactor library information in `arpdata.txt` in the `SCALE-6.2.3/data` directory and for reactor library files in `SCALE-6.2.3/data/arplibs/`
- Unless... `arpdata.txt` and library files exist in the SCALE temporary directory!
- You can copy your `arpdata.txt` and library files to the SCALE temporary directory using a `shell` command at the beginning of your Origami (or ORIGEN) input file:

```
=shell
cp path/to/files/myarpdata.txt  ${TMPDIR}/arpdata.txt
cp path/to/files/libraryfile1   ${TMPDIR}
cp path/to/files/libraryfile2   ${TMPDIR} etc...
end
=origami ...
```

# Problem Input Data

- ORIGEN reactor libraries for the two IRT assembly types are available in SCALE under the names
  - **irt4m6tube** for IRT-4M 6-tube 19.75%  $^{235}\text{U}$
  - **irt3m6tube90enrich** for IRT-3M 6-tube 90%  $^{235}\text{U}$
- Initial mass of U in assembly
  - assume 1MTU initial for convenience
  - results can be scaled to actual assembly U mass (e.g., U ~ 1.3kg for **irt4m6tube** assembly)
- Assumptions for each assembly type:
  - 10 operating cycles (30d each, with 30d downtime between cycles)
  - specific power 300MW/MTU

Sources of data for ORIGEN IRT reactor libraries:

1. A. Rakhmanov, J. R. Deen, N. A. Hanan, and J. E. Matos, "A Neutronic Feasibility Study for LEU Conversion of the WWR-SM Research Reactor and Uzbekistan", International Meeting on RERT, Sao Paulo, Brazil, ANL/TD/CP-9749, 1998.
2. P. L. Garner and N. A. Hanan, "Transient Analyses for the Tajoura Critical Facility with IRT-2M HEU Fuel and IRT-4M LEU Fuel: ANL Independent Verification Results", RERT Program, Argonne National Laboratory, ANL-05/58, 2005.
3. J. R. Deen, V. A. Hanan, J. E. Matos, P. M. Egorenkov, and V. A. Nasonov, "A Neutronic Feasibility Study for LEU Conversion of the IR-8 Research Reactor", International Meeting on RERT, Sao Paulo, Brazil, ANL/TD/CP-97537, 1998.

Let's work on this together!

