

Progress Towards Long-Pulse, High-Performance Advanced Tokamak Plasmas on the DIII-D Tokamak

by
M.R. Wade*

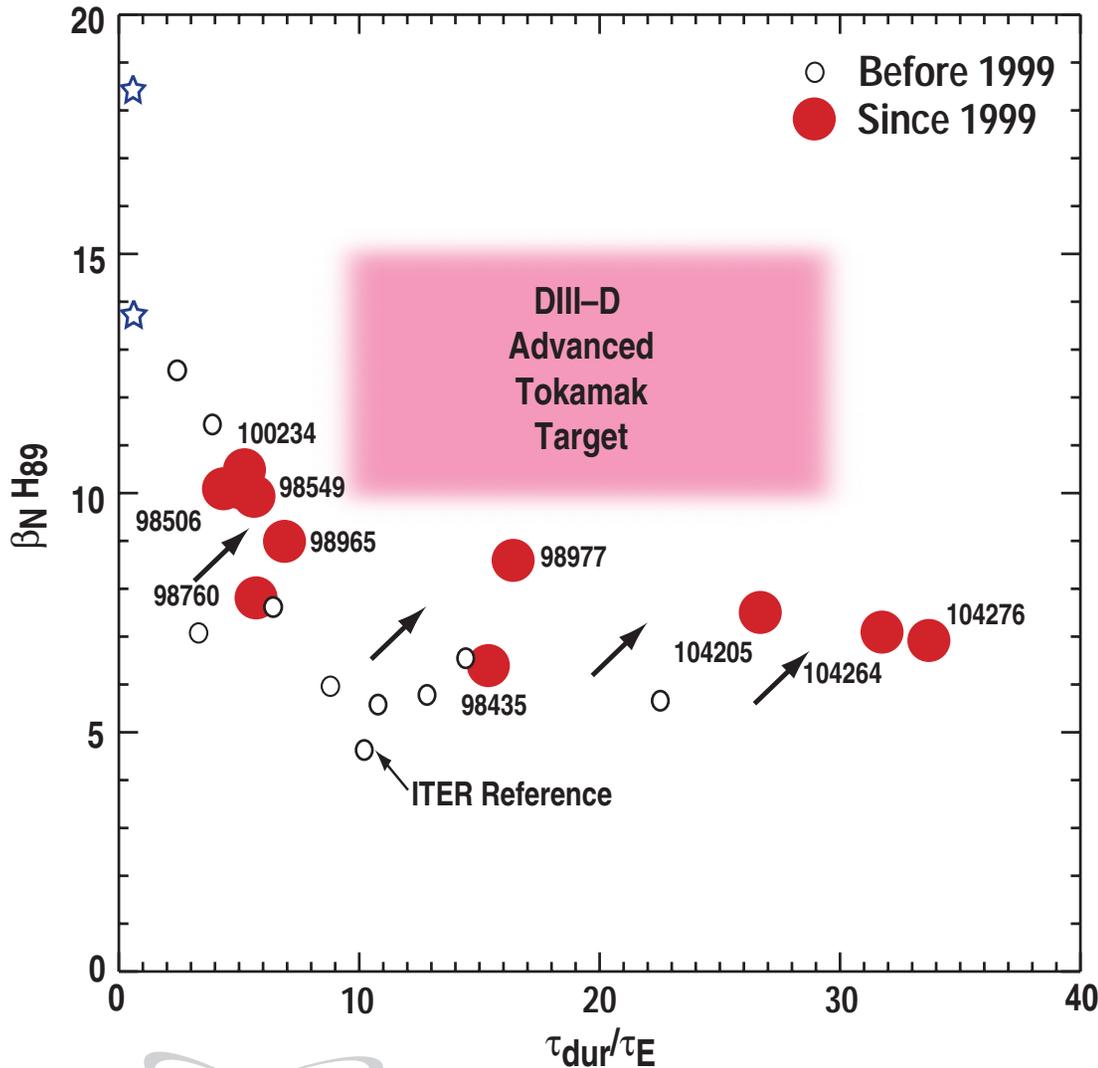
*Oak Ridge National Laboratory

Presented at
the American Physical Society
Division of Plasma Physics Meeting
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THE GOAL OF THE DIII-D ADVANCED TOKAMAK PROGRAM IS TO DEVELOP THE BASIS FOR A STEADY-STATE, HIGH PERFORMANCE TOKAMAK

- **Simultaneously require:**

- High fusion power density \Rightarrow High plasma pressure (high β)
- High fusion gain \Rightarrow Good energy confinement (high τ_E)
- Non-inductive current sustainment \Rightarrow High bootstrap fraction (high β_p)

- **Gain and bootstrap current have conflicting scalings**

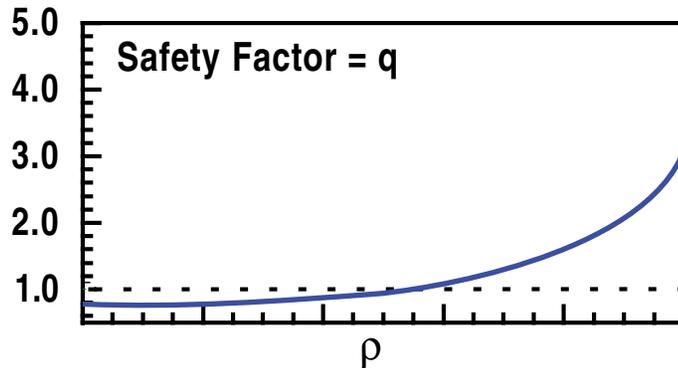
- Fusion gain: $\beta\tau_E \propto (\beta_N/q) (H_{89}/q^\alpha)$
- Bootstrap current: $f_{BS} \propto \beta_p \propto q \beta_N$

\Rightarrow Self-consistent scenarios require β_N and H_{89} above conventional tokamak values

Definitions: $\beta_N = \beta/(I/aB)$ $H_{89} = \tau_E / \tau_{E,ITER89P}$

COMPARISON OF CONVENTIONAL AND ADVANCED TOKAMAK FEATURES

Conventional



Features of Monotonic q Profile:

High gain

Moderate turbulence

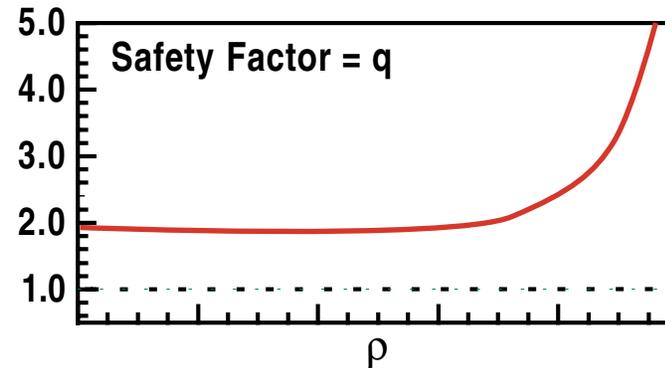
Resistive modes limit β

$$f_{BS} < 30 \% ; H_{89P} < 2.0 ; \beta_N < 2.5$$



Pulsed

Advanced



Features of High q_{min} and Low Shear:

High f_{BS}

Reduced turbulence

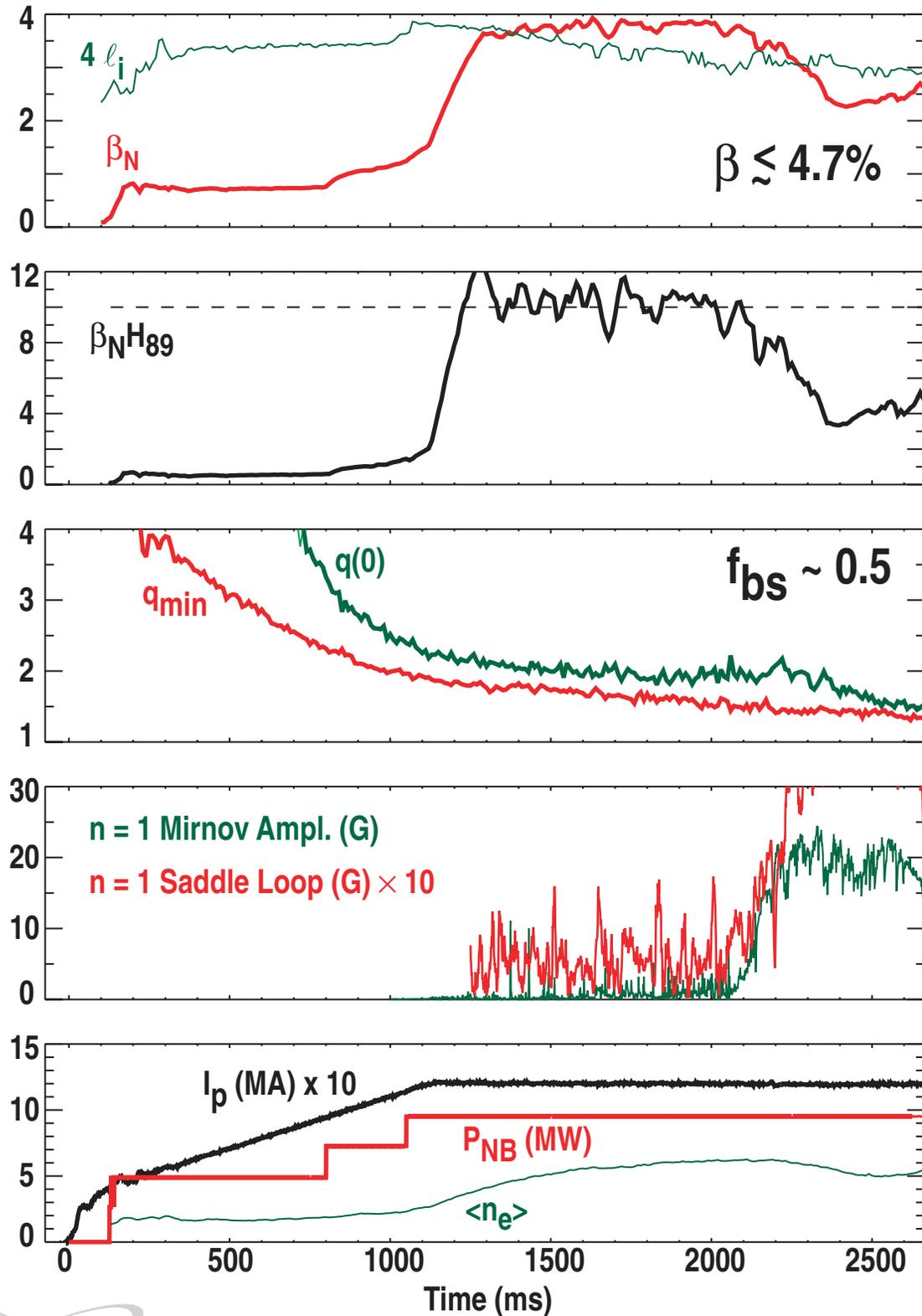
Ideal modes limit β

$$f_{BS} > 50 \% ; H_{89P} > 2.5 ; \beta_N > 3.5$$

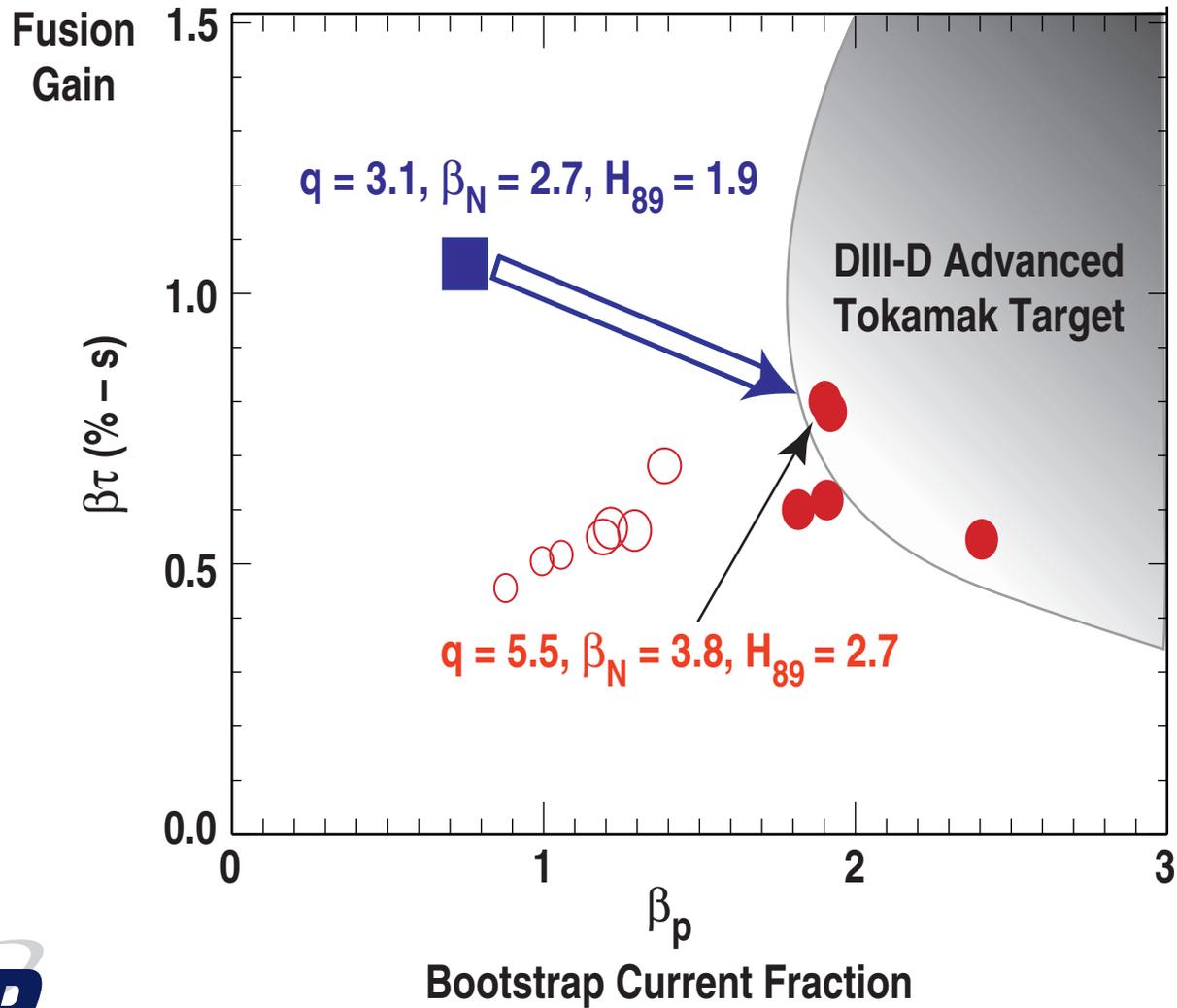


Potential for Steady-State

HIGH NORMALIZED PERFORMANCE (~ 10) SUSTAINED FOR $5 \tau_E$

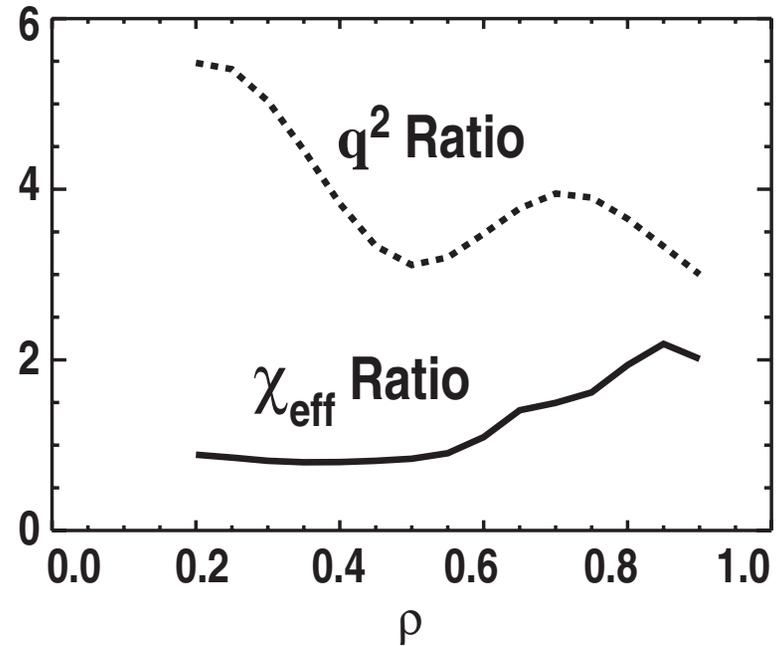
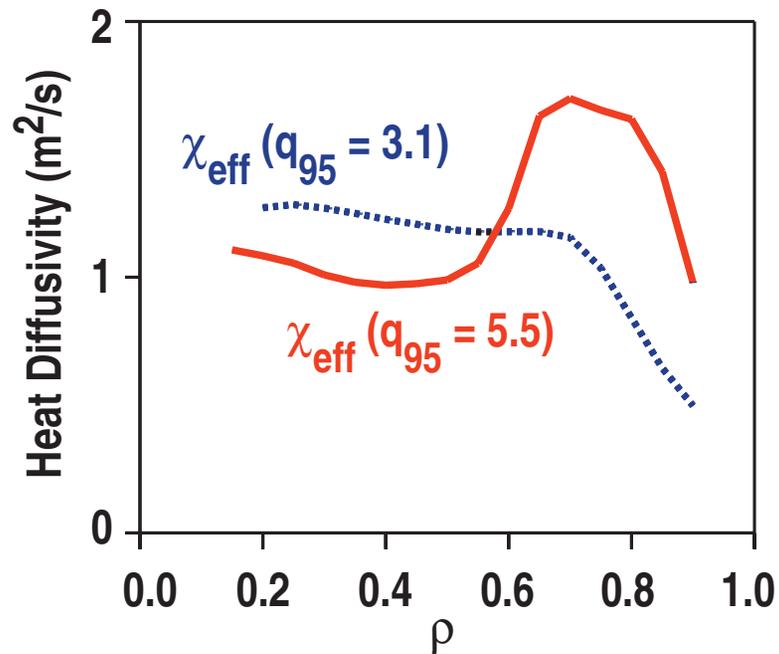


POTENTIAL FOR STEADY-STATE OPERATION IS ACHIEVED FOR MODERATE REDUCTION IN FUSION GAIN



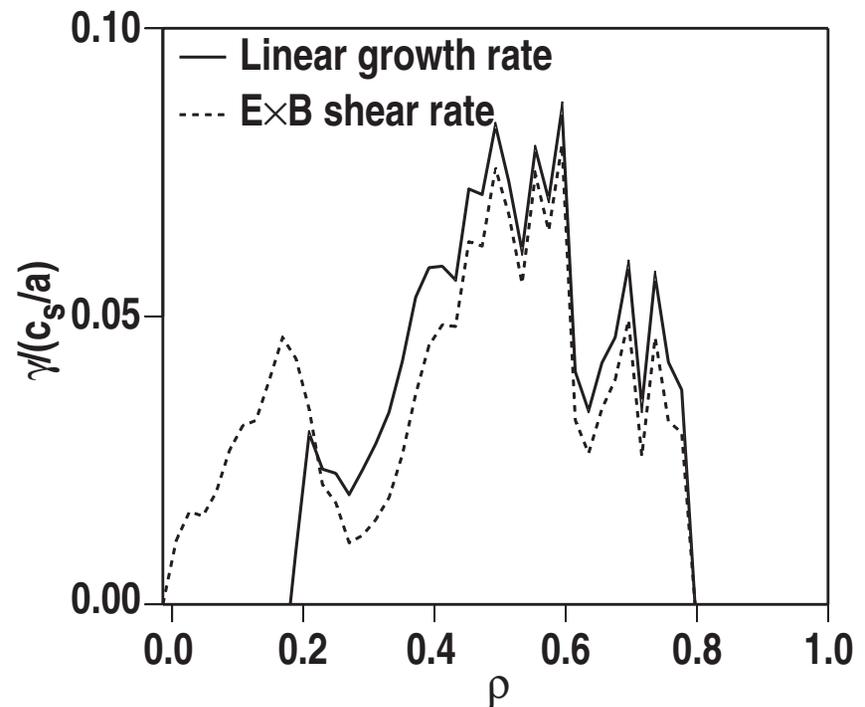
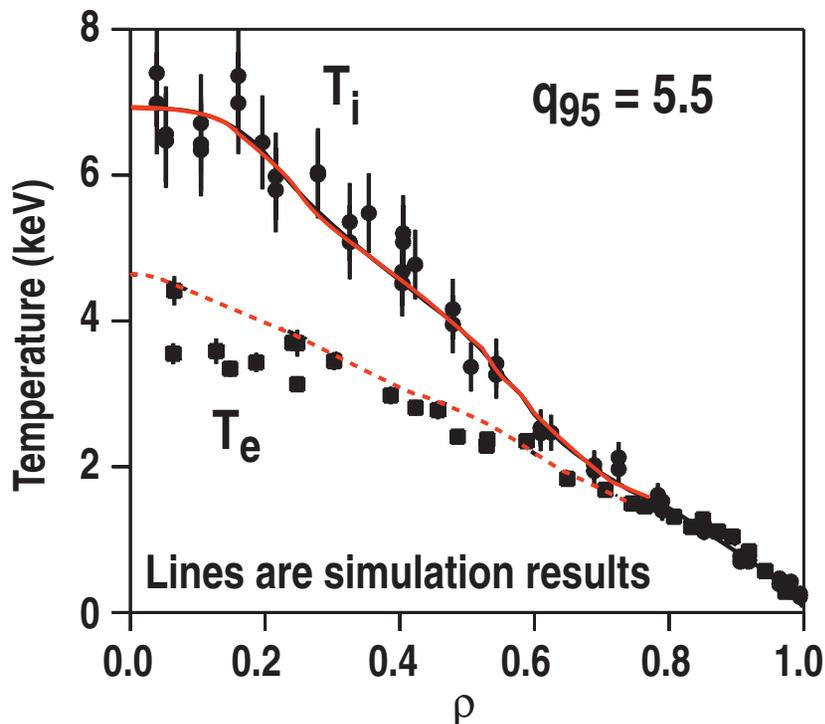
ENERGY TRANSPORT IS NOT SUBSTANTIALLY ALTERED BY THE CHANGE IN q PROFILE

- Neoclassical and empirical scalings predict $\chi \propto q^2$



IMPROVED CONFINEMENT IS CONSISTENT WITH DRIFT-WAVE SIMULATION WITH $E \times B$ SHEAR

- GLF23 model* contains ITG, TEM, and ETG with effects of $E \times B$ shear
- Self-consistent simulation shows reduction but not suppression of turbulence, consistent with measured $\chi_i > \chi_{i, neo}$

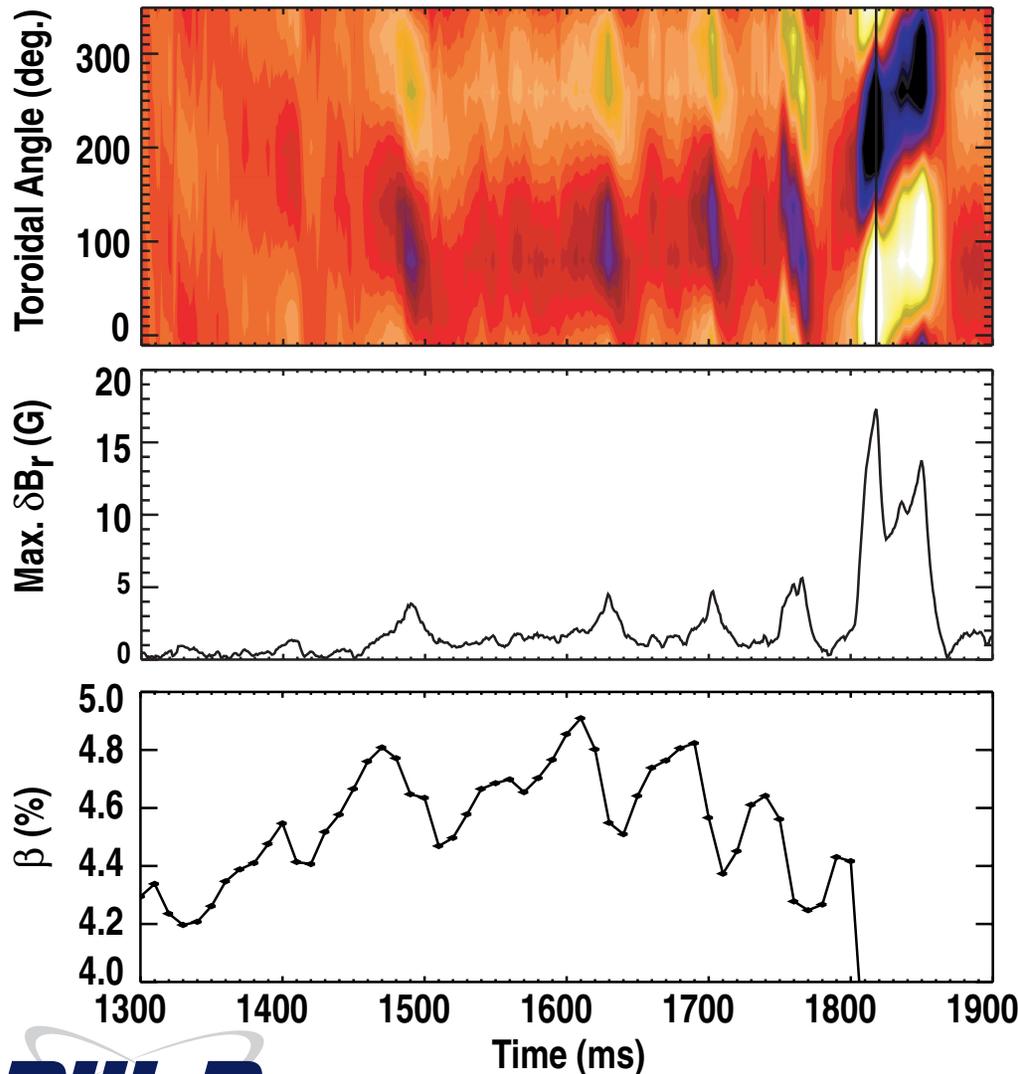


*Waltz et al, Phys. Plasmas 5 1695 (1998)

Politzer GP1.114

MAXIMUM β IS LIMITED BY RESISTIVE WALL MODES

δB_r measured by saddle coils outside the vessel

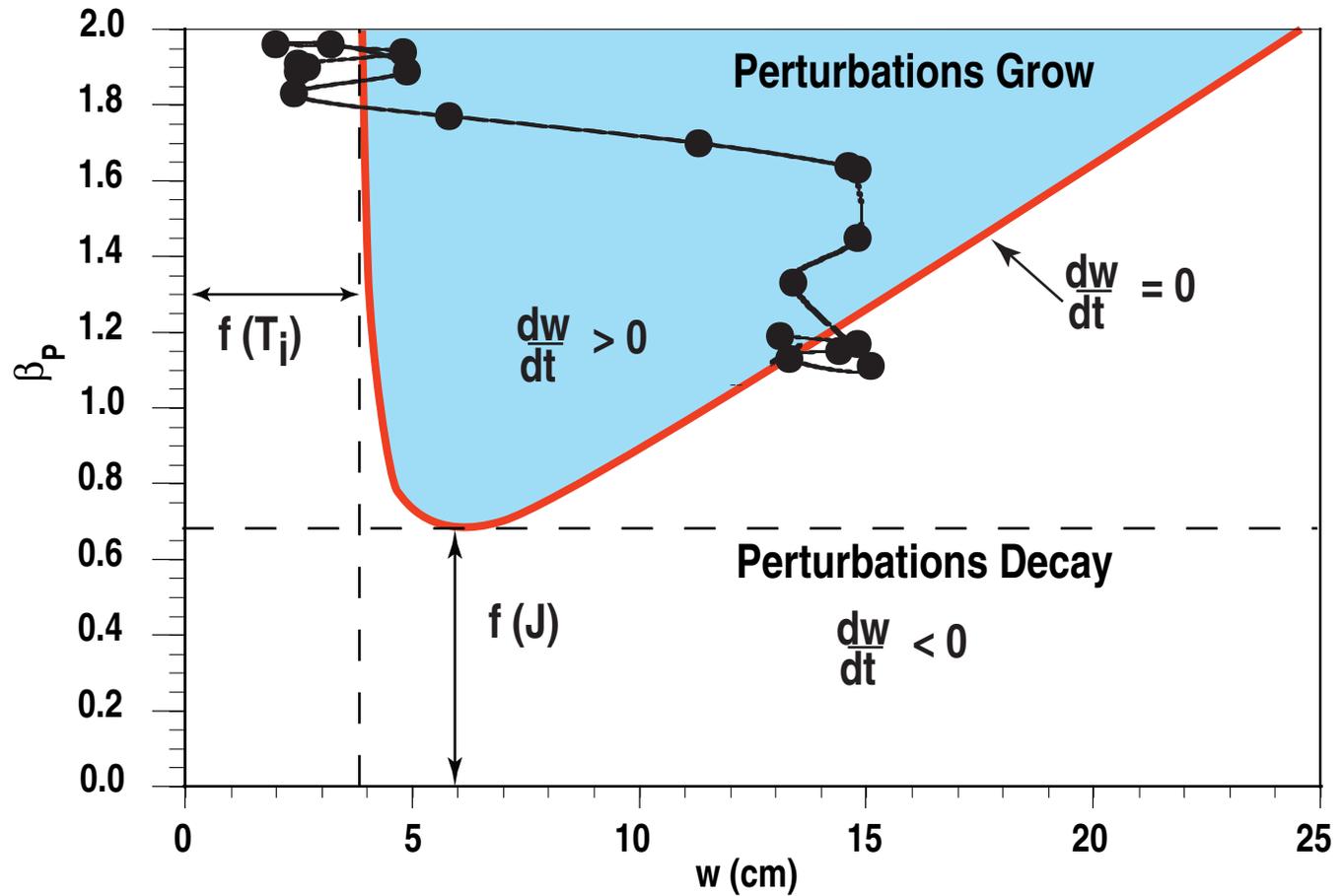


Limiting modes have the characteristics of resistive wall modes:

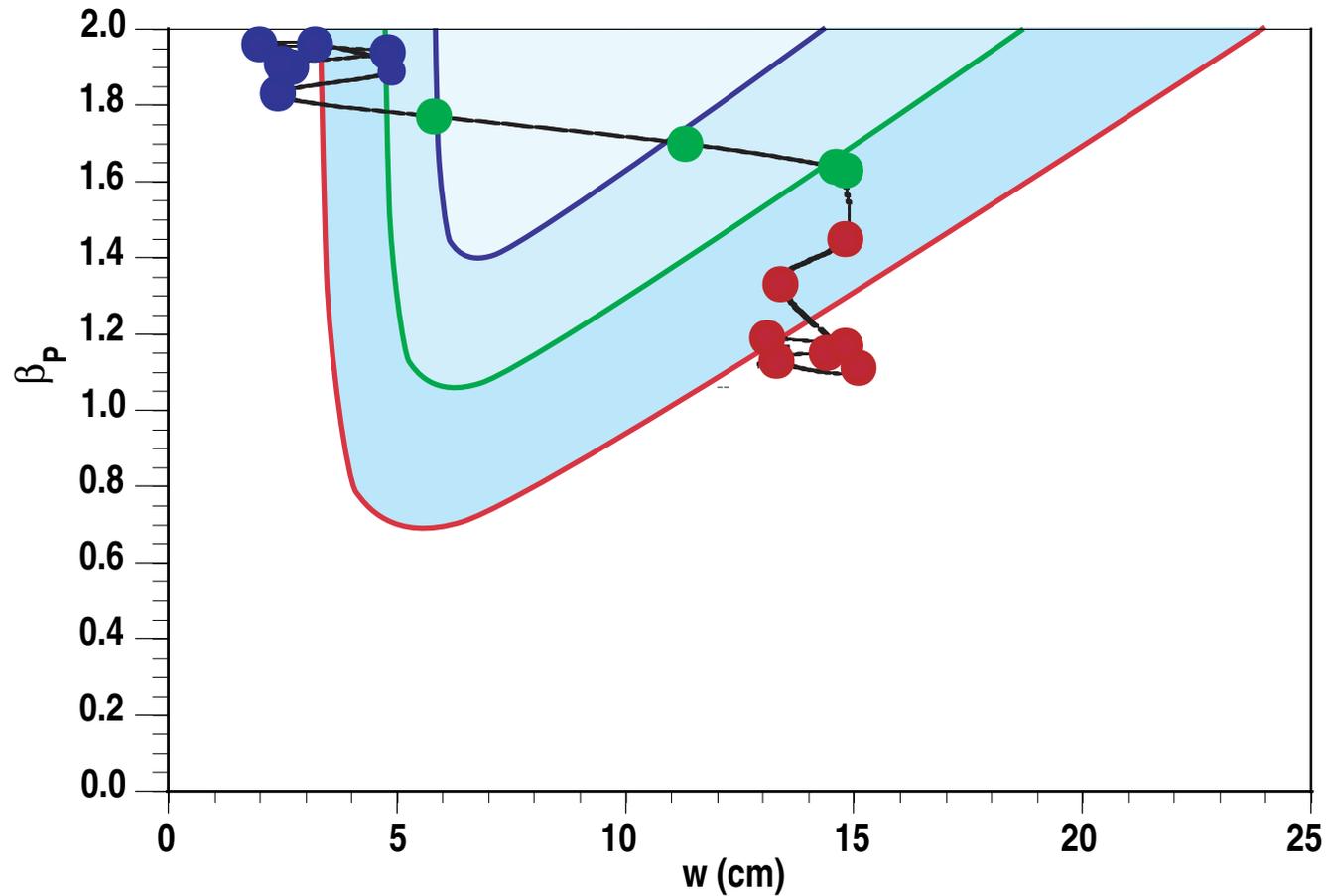
- Onset is at or above the no-wall ideal limit ($\beta_N \gtrsim 4l_j$)
- Growth rate consistent with characteristic wall time
- Real frequency (<100 Hz) consistent with wall time, not fluid rotation
- Proof of principle experiments on feedback control of the resistive wall mode indicate the possibility of raising the β limit

See M. Okabayashi GI1.5
Garofalo M01.004

TEARING MODE BEHAVIOR AGREES WITH NEOCLASSICAL TEARING MODE MODEL

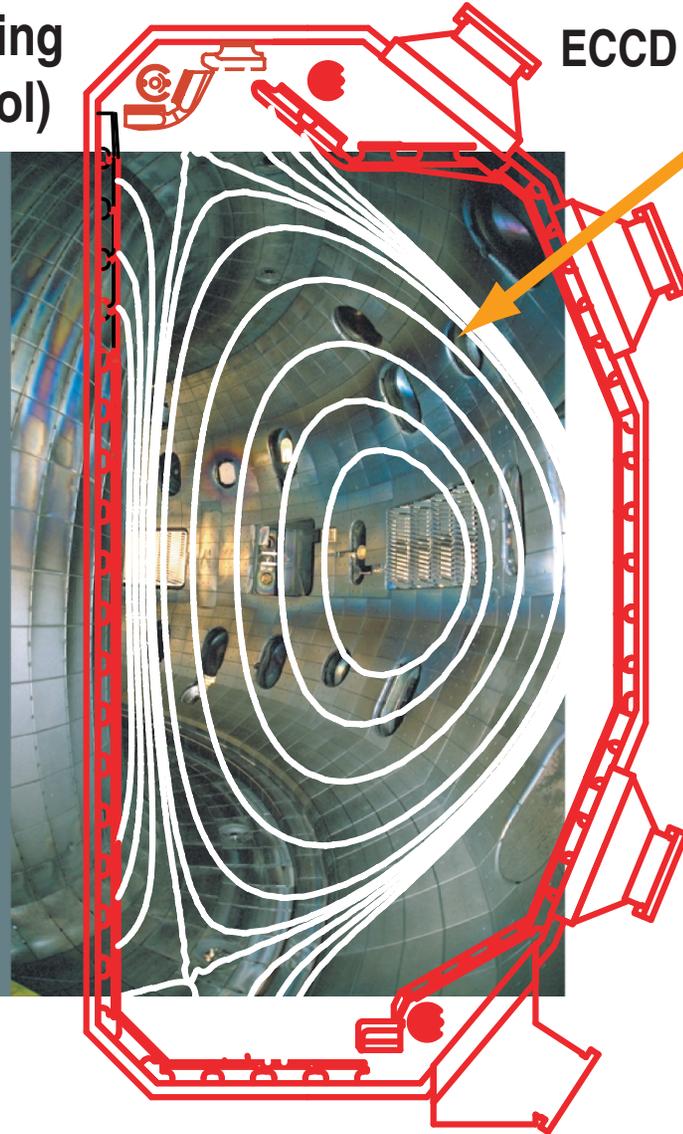


TEARING INSTABILITY MAY OCCUR DUE TO LACK OF CURRENT SUSTAINMENT AND DENSITY CONTROL



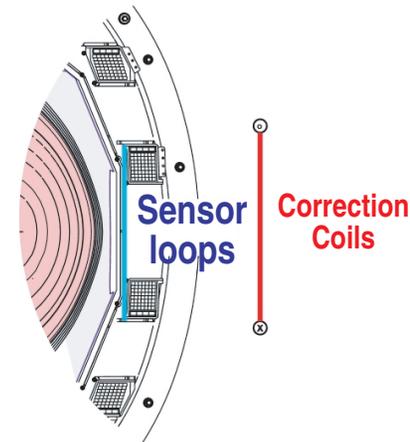
DEVELOPMENT OF CONTROL TOOLS IS NECESSARY TO EXPLOIT THE PHYSICS OF ADVANCED TOKAMAKS

Divertor Pumping
(Density Control)



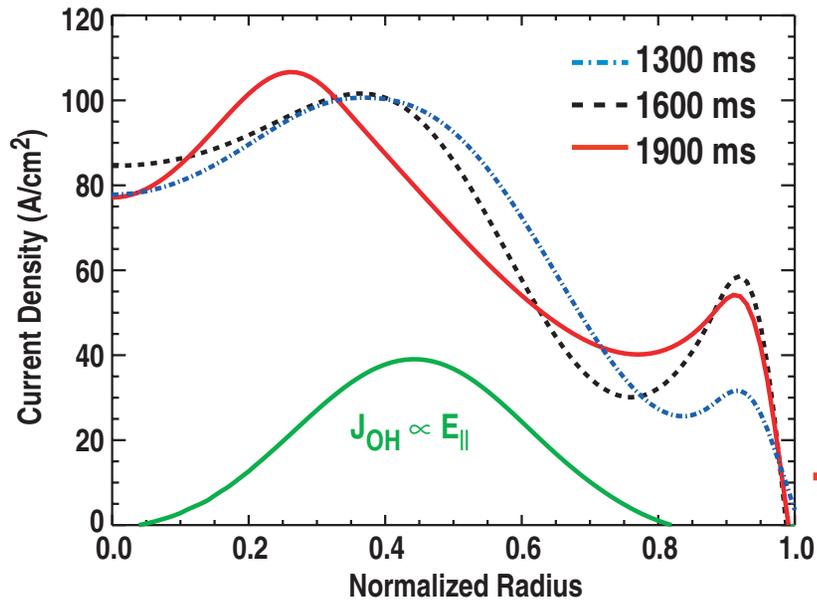
ECCD (Current Profile Control)

RWM Feedback
Control Coils ($\beta_N > \beta_N^{\text{no-wall}}$)

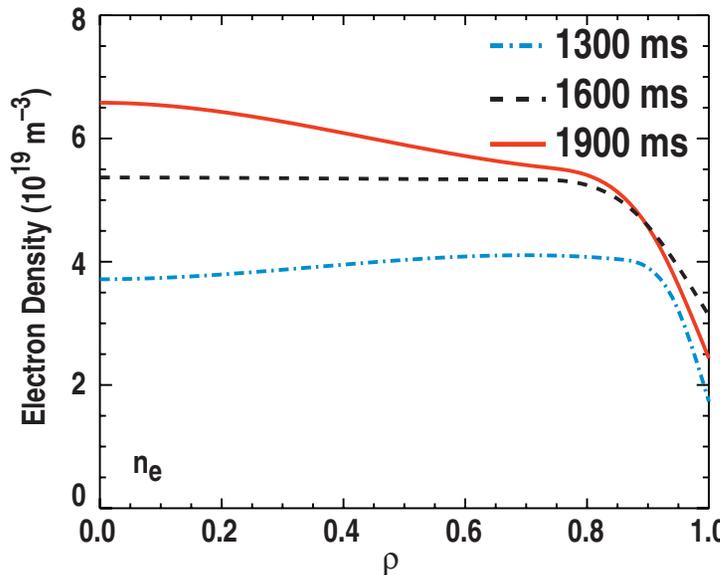
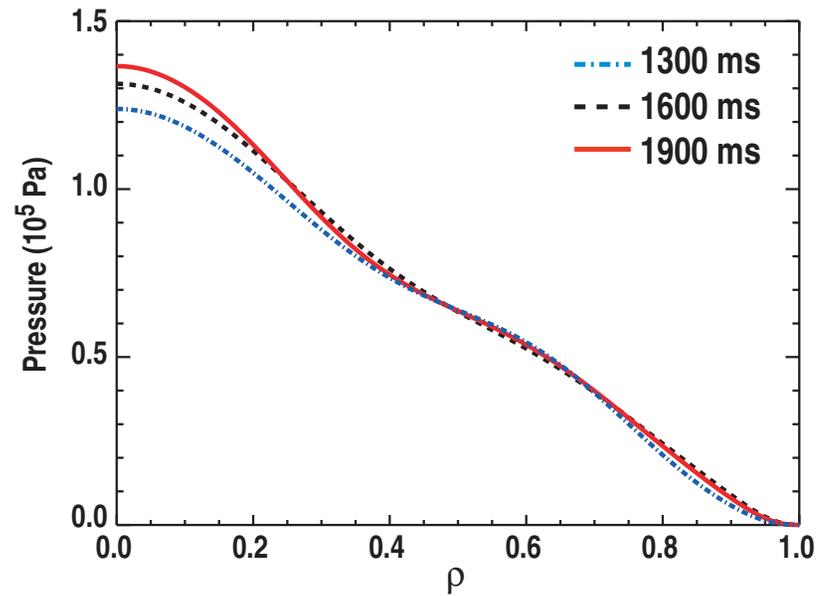


Plasma Shaping
($\beta_N > 3.0$)

CURRENT PROFILE EVOLVES THROUGHOUT THE HIGH PERFORMANCE PHASE AT CONSTANT PRESSURE



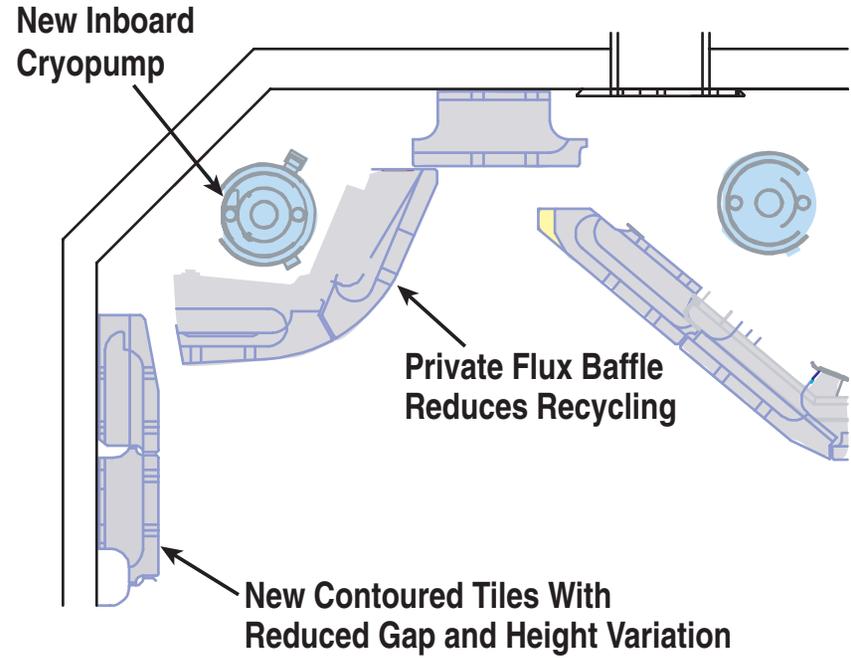
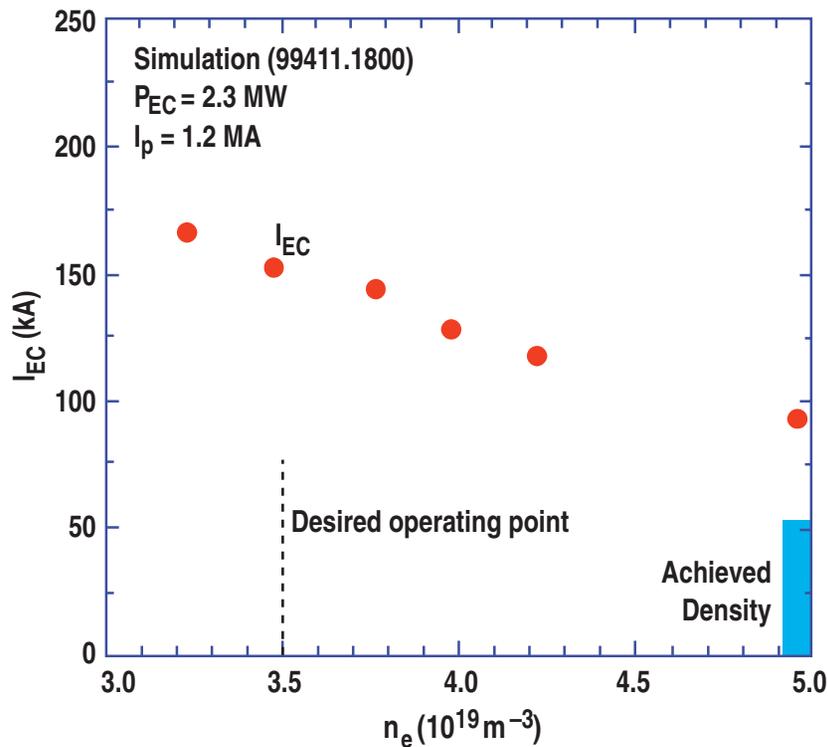
Off-axis ECCD Required



⇒ Density control is required

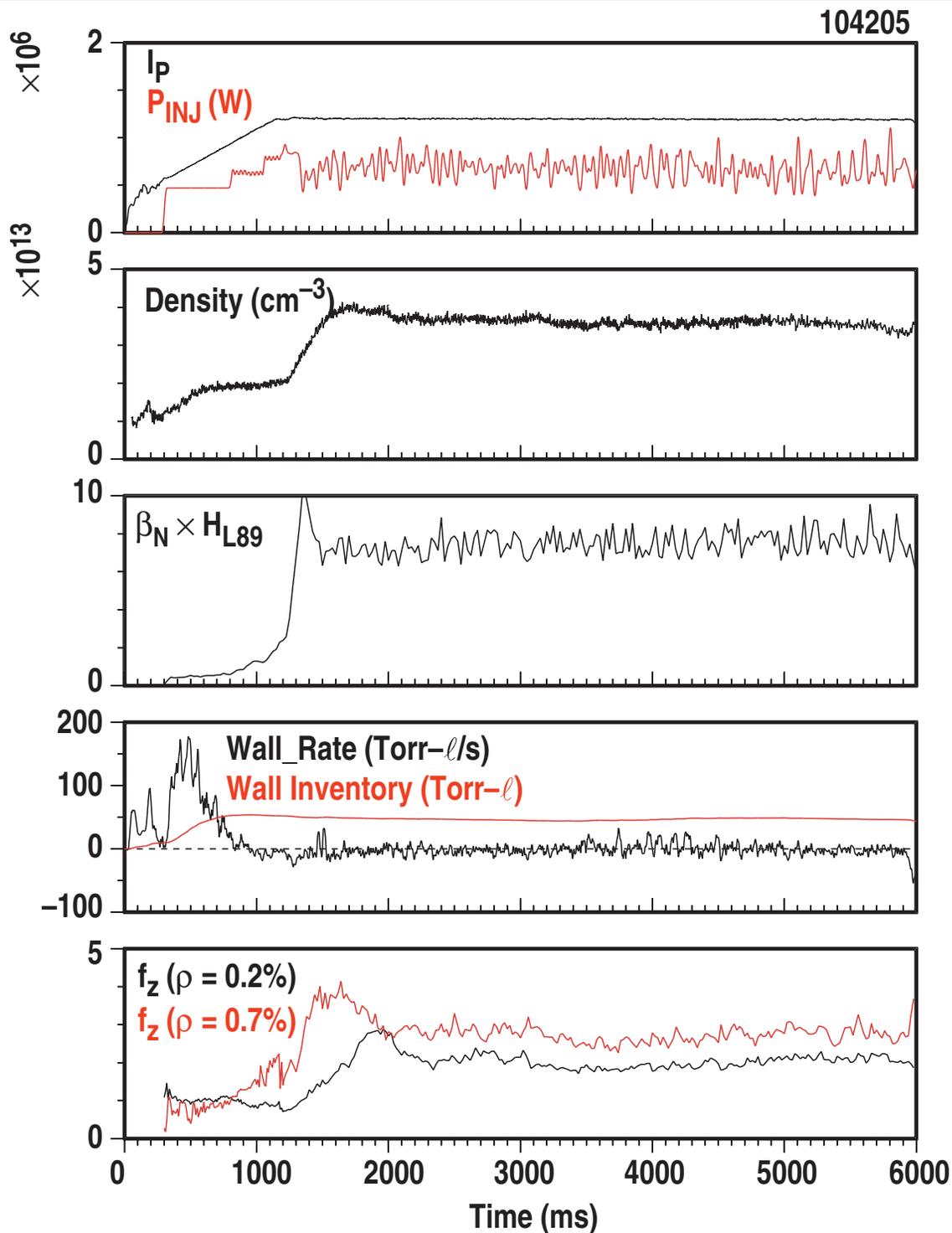
DENSITY CONTROL WILL MAXIMIZE THE EFFECTIVENESS OF OFF-AXIS ECCD

$$I_{EC} \propto \frac{T_e}{n_e} \frac{1}{(Z_{eff} + 5)} \Rightarrow I_{EC} \propto \frac{1}{n^2} \text{ at constant } \beta$$

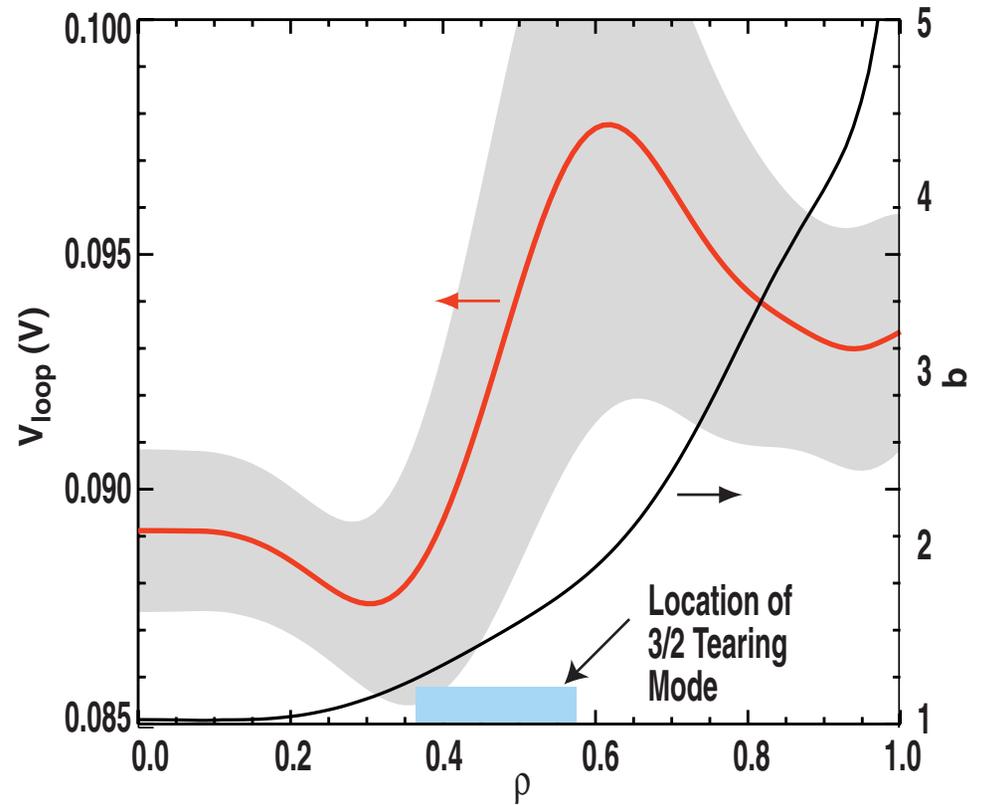
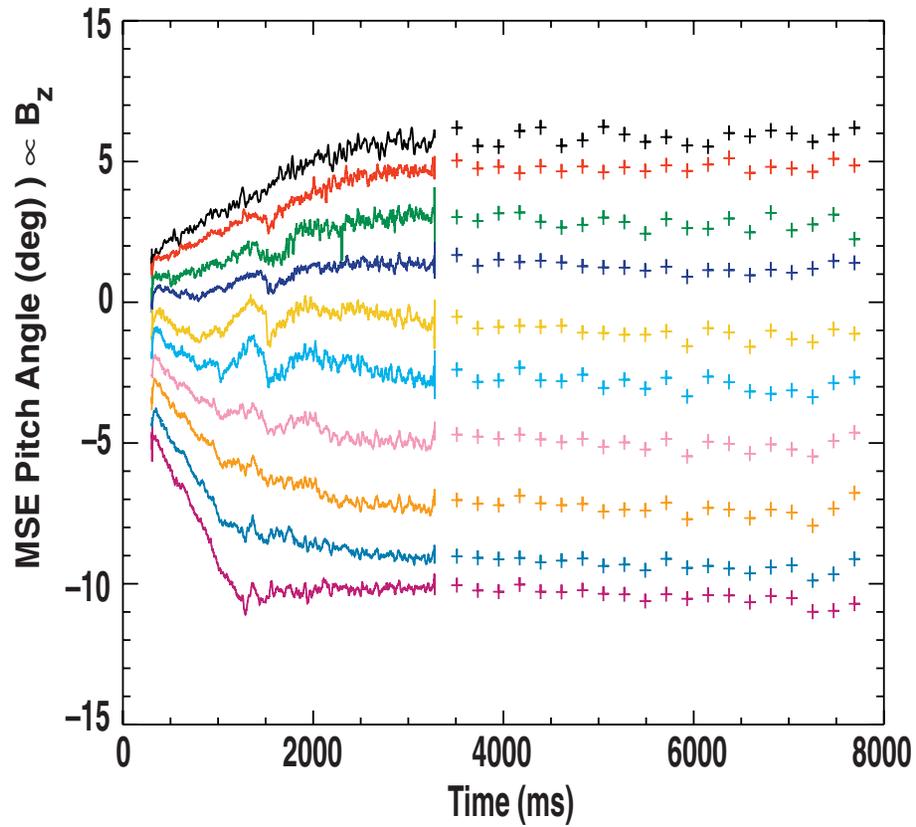


- New divertor allows pumping in high triangularity shape

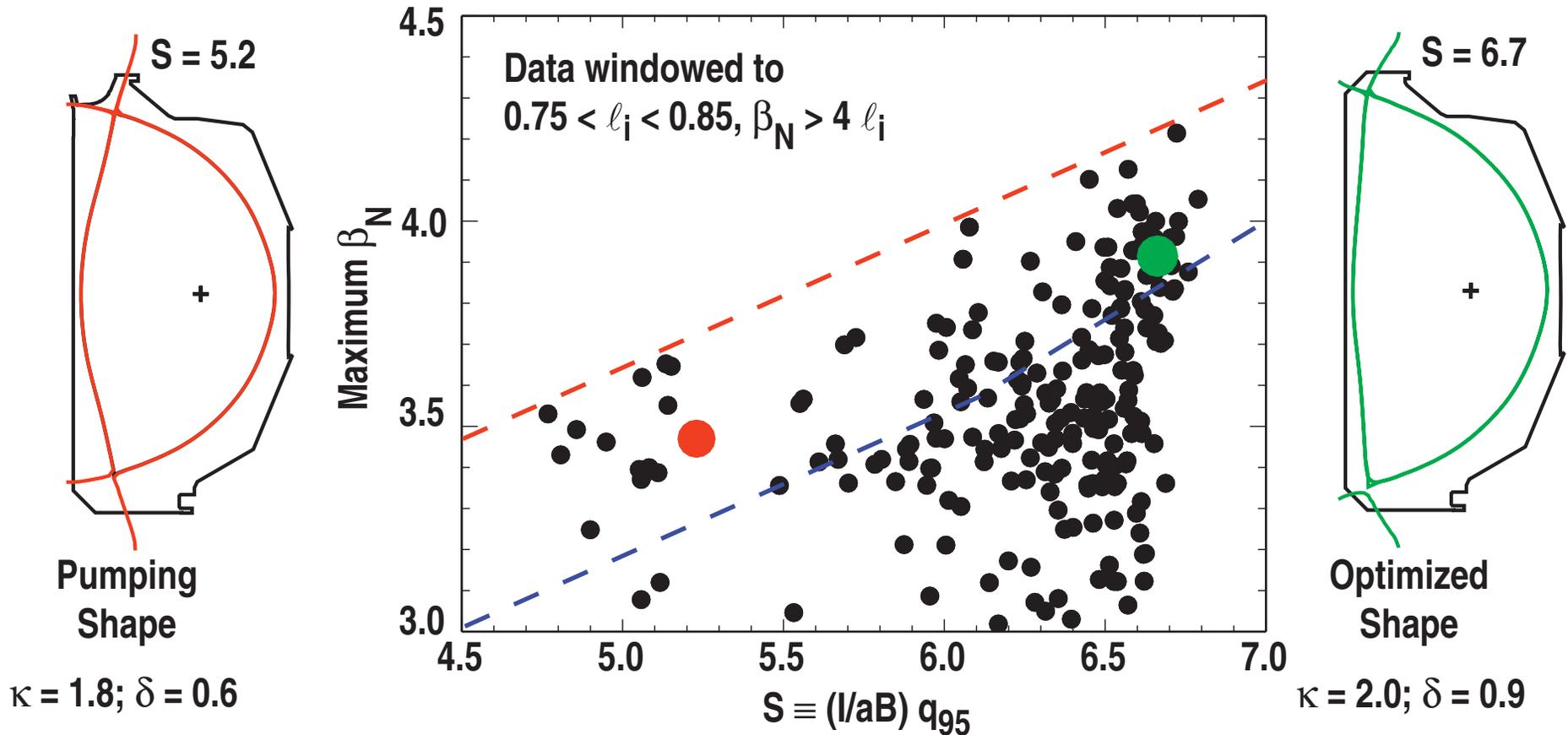
DENSITY AND IMPURITY CONTROL HAS BEEN DEMONSTRATED IN LONG-PULSE ELMING H-MODE DISCHARGES WITH $\beta_N H_{89p} \sim 7.5$ FOR OVER $25 \tau_E$



INTERNAL MAGNETIC MEASUREMENTS INDICATE THAT THE CURRENT PROFILE IS NOT EVOLVING



RWM β LIMIT HAS A SIGNIFICANT DEPENDENCE ON PLASMA SHAPE

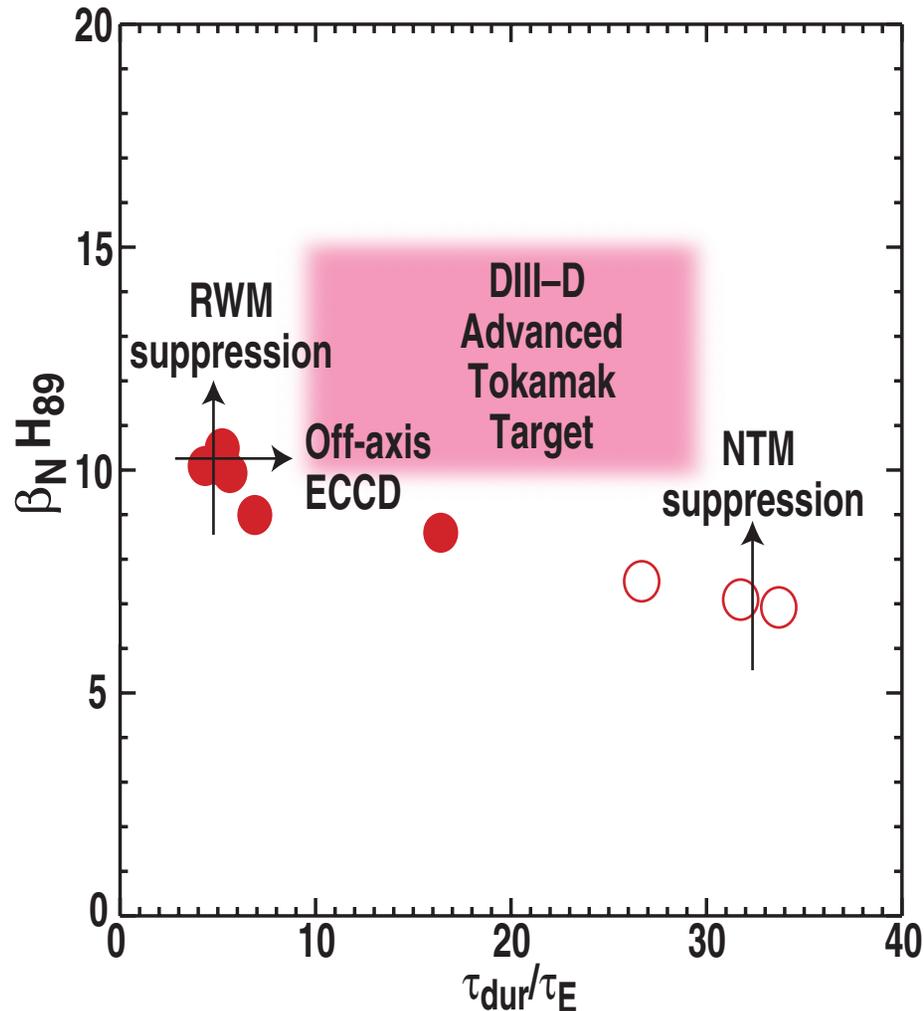


- Experimental scans at constant I/aB
- Stability calculations at constant q

SUMMARY

- **Substantial progress has been made in the development of long-pulse advanced tokamak scenarios**
 - $\beta_N H_{89} \sim 10$ for $5 \tau_E$
 - $\beta_N H_{89} \sim 9$ for $16 \tau_E$
- **Stability**
 - Resistive wall modes are the β limiting instability in most discharges with $q_{\min} \geq 1.5$
 - Neoclassical tearing modes limit β in discharges with $q_{\min} \sim 1$ and sometimes limit the duration of higher q_{\min} discharges
- **Confinement**
 - Local heat diffusivity on high q_{\min} plasmas similar to that found on conventional sawtoothed H-mode plasmas
 - Electron and ion temperature profiles are well simulated by an ITG model including $E \times B$ shear
- **Current evolution**
 - Non-inductive current fraction is 60%–75% in high q_{\min} discharges
 - Remaining inductive current is peaked off-axis
- **Control tools**
 - Density and β control demonstrated by operating at $\beta_N H_{89} \sim 7$ for 6.3 s with β at >90% of the 2/1 tearing mode limit

EXTENSION OF HIGH PERFORMANCE RESULTS RELY ON MITIGATION OF RESISTIVE MHD MODES (RWMs and NTMs)



High Bootstrap Fraction Discharges

- β_N limited by resistive wall modes (RWMs)
⇒ Need feedback stabilization
- Duration limited by current evolution
⇒ Need off-axis ECCD

Long-Pulse, High-Performance Discharges

- β_N limited by neoclassical tearing modes
⇒ Need NTM stabilization