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**A STUDY OF THE FISSION PRODUCT
RELEASE FROM A BADLY DAMAGED
WATER-COOLED REACTOR**

(Thesis)

D. Y. Hsia

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A STUDY OF THE FISSION PRODUCT RELEASE FROM A BADLY
DAMAGED WATER-COOLED REACTOR

D. Y. Hsia and R. O. Chester

Submitted by D. Y. Hsia to the Graduate School
of the University of Tennessee
in partial fulfillment of the requirements
for the degree of
Master of Science

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Oak Ridge, Tennessee 37830
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ABSTRACT

A realistic model for fission product release from a badly damaged power reactor has been developed in this study. The vaporization of fission products by decay heat, the effects of the metal-water reaction, the deposition of particulate inventory in reactor vessels, and the leakage through the multiple containments are all considered in this model.

The calculation shows that the vaporization of fission products can lead to a 35 percent reduction of the decay heat retained within the core. This effect is greatest for low-vapor-pressure fission products. The metal-water reaction heat available to vaporize UO_2 is about 10 percent of the decay heat available. The reaction heat can produce a reducing atmosphere surrounding the fuel leading to an increase of Ba and Sr releases by a factor of 30 or more. The effect of containment on the fission product release depends on the deposition properties of the specific fission product. But even a badly damaged containment can lead to a reduction of the fission product release by more than two orders of magnitude. A recirculating filter-absorber inside the containment can lead to a reduction of the particulate fission product release by more than one order of magnitude.

CHAPTER I
INTRODUCTION

The objective of this study was to develop a model which could relate the accidental fission product release from a power reactor to the reactor configuration and postaccident situation.

The sequence of events following the reactor accident (Ergen, 1967) was summarized briefly in Table 1. Following the double-ended rupture of a large, primary-system pipe, there was a violent blowdown which took about ten seconds. Then the core temperature began to rise due to the decay heat, and the various fission products started to vaporize after 30 to 50 seconds. When the reactor core reached temperatures greater than 1100°C , the zirconium-steam reaction began to become a significant energy source. At the same time the vaporized fuel and fission products were released into the pressure vessel, and if they did not deposit, leaked into the various containments and finally into the environment. The pressure vessel melt-through was expected in from 30 to 120 minutes after the pipe rupture. The fission product release would continue after the pressure vessel melt-through, but it would be influenced by the interaction of the fuel with the containment atmosphere and materials presented in the containment.

In this study a vaporization model (Appendix VI of Auxier and Chester, 1973) was used to calculate the vaporization rates of various fission products, and an aerosol model (Davis, 1971) was used to calculate the deposition of particulate fission products.

TABLE 1
SEQUENCE OF EVENTS FOLLOWING THE REACTOR ACCIDENT

Time	Event	Cause
0	Reactor shutdown	Damage to the reactor vessel, to the reactor primary cooling system, and to the emergency core cooling systems. A double-ended, primary-coolant-system pipe break is assumed.
10 secs	Reactor blowdown completed	Rapid depressurization of pressure vessel and coolant systems
30-50 secs	Fission products and UO_2 start to vaporize	Fission product afterheat
40 sec-30 min	Zr-steam reaction	Fission product afterheat
30-120 min	Possible pressure vessel melt-through	Fission product afterheat and Zr-steam reaction heat

After the rapid depressurization and loss of coolant following damage to the reactor core and cooling system, some of the more volatile fission products were among the most energetic heat sources (Morrison et al., 1966). Since these fission products had higher vapor pressure, most of them vaporized from the core shortly after the accident. Hence, the decay heat available for vaporizing the remaining fission products is reduced. In this study this negative feedback effect, which could lead to a reduction of the releases of the less volatile fission products, was investigated.

In recent years, there has been more concern about the zirconium-steam reaction because of the increasing use of zircaloy as a cladding material and because of the use of a water spray as an emergency safety device. Hence, there were many papers dealing with this effect (see, for example, Baker and Ivins, 1965; Dietz, Nyer and Wilson, 1966; Morrison et al., 1966). But almost all of these reports emphasized the effects of zirconium-steam reaction on core temperature rather than the vaporization rate of fission products. In this study the magnitude of zirconium-steam reaction heat relative to the decay heat and its effect on fission product vaporization were investigated.

After a reactor accident, containment was the most important safety device to reduce the fission product release to the environment. In this study, the effect of multiple containments and the degree of damage to the containments on the fission product release was investigated.

CHAPTER II

DESCRIPTION OF THE FISSION PRODUCT RELEASE MODEL

I. BASIC ASSUMPTION OF THE VAPORIZATION MODEL

First, it was postulated that the rate of fission product vaporization from the reactor core was proportional to the sum of the fission product decay heat rate and zirconium-steam reaction heat. Secondly, it was postulated that UO_2 was vaporized by a constant fraction of the decay heat and a different constant fraction of the zirconium-steam reduction heat from 60 seconds after shutdown onward. The effect of fission product vaporization on decay heat generation was included and will be discussed in detail in Chapter III. Thirdly, it was postulated that each fission product vaporization rate was proportional to the ratio of the vapor pressure of the fission product to that of UO_2 . The vaporization rate was assumed to be zero during a heatup period of ten seconds and to increase linearly from zero at ten seconds after shutdown to the value at 60 seconds. The ten-second heatup period assumption was taken from the core thermal history after a loss of coolant accident (Ergen, 1967) as mentioned in Chapter I.

The three postulates were reasonable. The third postulate was justified by R. J. Davis' work (Appendix VI of Auxier and Chester, 1973). He compared the calculated vaporized fraction of the core inventory of fission product with the observed inventory for several isotopes. The agreement was reasonably good. The second postulate was made for simplicity and the effects of changing this postulate

are examined in this thesis. The first postulate states that the total heat source being considered is from two sources.

II. BASIC ASSUMPTION OF THE AEROSOL MODEL

There are many aerosol models reported in recent years. R. J. Davis' model (Davis, 1971) was adopted in this study because it was simple but reasonably accurate.

Loss of airborne material in a container was assumed to occur due to agglomeration and settling of aerosol particles. In the settling process a time-independent particle size distribution was assumed to describe the complete size spectrum from a minimum particle volume to a maximum particle volume. The maximum and minimum particle volumes were also assumed to be independent of time. These particles were presumed to be completely mixed so that the concentration was everywhere the same except in a stagnant boundary layer along the floor. Particles that fall into the boundary layer did not return. The settling velocity was obtained by using Stokes' law together with the assumed particle size distribution. By a mathematical analysis, the following differential equation was obtained to describe the settling process.

$$\frac{dn(t)}{dt} = -K_s (V_{max}, V_{min}, \rho, h, \eta, \lambda, a, g) n(t) \quad (1)$$

where $n(t)$ is the total particle number concentration at time t , K_s is a settling rate constant, V_{max} , V_{min} are the maximum and minimum particle volumes respectively, ρ is the particle density, h is the height of the container, η is the viscosity of the air, λ is the mean free path of the molecules of the gaseous medium, a is the Cunningham correction constant, and g is the acceleration due to gravity.

The kinetics of the agglomeration process have been worked out by Smoluchowski (1916, 1917). Additional mathematical treatment of the theory has been developed by Fuchs (1934) and Chandrasekhar (1943). It was presumed that any particle that becomes larger than V_{max} by coagulation falls instantaneously. In other words, they were presumed to disappear at the same rate as they are formed by coagulation. Hence, the decrease in total number concentration by this process is

$$dn(t)/dt = -K_c n^2(t) \quad (2)$$

where K_c is a coagulation rate constant. An additional expression can be written for the formation of particles:

$$dn(t)/dt = S(t) \quad (3)$$

where $S(t)$ is the rate of aerosol formation which is obtained by the vaporization model. Hence, the equation for the change in aerosol particle number concentration with time has the form

$$dn(t)/dt = -K_c n^2(t) - K_s n(t) + S(t) \quad (4)$$

III. MATHEMATICAL FORMULATION

List of Symbols

- B total inventory of one fission product airborne in the reactor building, curies
- BWR Boiling water-cooled reactor
- C total inventory of one fission product airborne in the containment, curies
- E_b fractional removal efficiency of the reactor building

	filter-absorber for a PWR
E_C	fractional removal efficiency of the containment filter-absorber for a PWR
F'	ratio of vapor pressures: fission product/ UO_2
F_B	flow rate through the reactor building filter absorber, cm^3/sec
F_C	flow rate through the containment filter absorber, cm^3/sec
F_U	fraction of core which has vaporized
H	fraction of fission product decay heat available to vaporize UO_2
H'	decay heat correction factor
H_{sub}	heat of sublimation of UO_2 , joules/g
K'_C	constant in aerosol agglomeration term, $cm^3 \text{ curie}^{-1} \text{ sec}^{-1}$
K_{sp}	constant in aerosol settling term (in pressure vessel), sec^{-1}
K_{sc}	constant in aerosol settling term (in containment), sec^{-1}
K_{sb}	constant in aerosol settling term (in reactor building), sec^{-1}
K_{st}	constant in aerosol settling term (in access tunnel), sec^{-1}
L_b	leakage rate of gas between the reactor building and the next containment, cm^3/sec
L_c	leakage rate of gas between the containment and the reactor building, cm^3/sec
L_p	leakage rate of gas in and out of the pressure vessel, cm^3/sec
L_t	leakage rate of gas between the access tunnel and the environment, cm^3/sec

M_o	M_u at reactor shutdown, g
M_u	mass of the core, g
P	total inventory of one fission product airborne in the pressure vessel, curies
P'	total amount of heat available to vaporize UO_2 , watts
P_{dh}	fission product decay heating rate, watts
P_o	reactor operating power level, watts
PWR	pressurized water-cooled reactor
Q	inventory of one fission product in the fuel, curies
Q_o	Q at reactor shutdown, curies
R	total inventory of one fission product airborne in the environment, curies
t	time after shutdown, sec
T	total inventory of one fission product airborne in the access tunnel, curies
V	rate of one fission product vaporization from the core, curies/sec
V_b	volume of the reactor building, cm^3
V_c	volume of the containment, cm^3
V_p	volume of pressure vessel, cm^3
V_t	volume of the access tunnel, cm^3
V_u	rate of vaporization of UO_2 , g/sec
λ	one fission product decay constant, sec^{-1}

Heat Generation

The fission product decay heating rate is taken to be approximately that for a core which has operated for infinite time (Remley, 1973),

$$P_{dh}(t) = 0.140 P_0 t^{-0.291} \quad (5)$$

Equation (5) is empirical and is approximately correct in the time range from 60 seconds after shutdown to 1 year after shutdown. Under the condition that all the fission products remain in the core.

The magnitude of the heat generated by zirconium-water reaction is estimated in Chapter IV. Although the heat generated by zirconium-water reaction is significant, its affect on the fission product release is much less than that of decay heat due to several other reasons which will be discussed later. Hence, the metal-water reaction heat is neglected in the present formulation.

Vaporization of UO₂

During the first 10 seconds after cessation of fissioning, no vaporization was assumed to occur

$$P' = 0 \quad \text{for } t < 10. \quad (6)$$

It was assumed that heatup of the core takes about one minute. Therefore, from 60 seconds after shutdown onward, it was assumed that a constant fraction H of the decay heat generated in that fraction of the core which had not yet vaporized, went to vaporize UO₂. Hence,

$$P'(t) = 0.140 H H'(t) P_0 t^{-0.291} \quad \text{for } t > 60 \quad (7)$$

For times between 10 and 60 seconds after shutdown, it was assumed that the decay heating to cause vaporization of UO_2 increased linearly from zero at 10 seconds to the heat rate given in Equation (5) at 60 seconds. Hence,

$$P'(t) = 0.140 \times 60^{-0.291} H H'(t) P_0 (t-10)/50$$

for $10 \leq t \leq 60$ (8)

The H value in Equations (7) and (8) depends on the heat transfer properties of the various material in the core, the geometric and mechanical situations after the accident and the amount of the emergency cooling water available. This value can not be predicted before the accident and is difficult to be determined even after a reactor accident has started. The effect of this value on the fission product release will be discussed in the next section of this chapter.

The H' value in Equations (7) and (8) depends on the fuel material and also on the assumed H value. In Chapter III, a method to calculate H' as a function of time is described and discussed. The result shows that H' is not sensitive to the change of the assumed H value.

The rate of vaporization of UO_2 is

$$V_u = P'/\Delta H_{sub} \quad (9)$$

The heat sublimation of UO_2 at $1800^\circ K$ is 137.1 kcal/mole (Belle, 1961) or 2150 joules/g. V_u is the rate of decrease in the mass of the core, M_u . Hence,

$$-V_u = dM_u/dt \quad (10)$$

From Equations (7), (8), (9), and (10), M_u can be obtained as a function of time by numerical methods.

Fission Product Vaporization and Inventory

The rates of vaporization of fission products are assumed to be proportional to the fractional rate of vaporization of UO_2 , the ratio of vapor pressure of the fission product to that of UO_2 , the amount of fission product not yet vaporized. Hence,

$$V = V_u F' Q / M_u \quad (11)$$

In general, the inventory of any fission product in the fuel will be given by integration of an equation of the form

$$dQ/dt = -V - \lambda Q \quad (12)$$

solving for Q and integrating gives

$$Q/Q_0 = \exp(-\lambda t) (M_u/M_0)^{F'} \quad (13)$$

Release of Vaporized Fission Products

The vaporized fuel and fission products were released into the pressure vessel and, if they did not deposit, found their way into the containment, the reactor building, any access tunnel associated with the power plant and finally the environment.

The inventory in the pressure vessel increased due to vaporization from the core at a rate of V curies per second. The inventory will decrease due to leakage from the pressure vessel to the containment and radioactive decay. A leakage flow proportional to the density of fission products from the pressure vessel to the containment and a similar flow from the containment to the pressure vessel were assumed.

Loss of particulate fission products is also due to agglomeration and settling of aerosol particles. It was assumed that particulate fission products would be associated with UO_2 particles and therefore that the fractional rate of deposition of particulate fission products would equal that of UO_2 particles. The "aerosol model" described in the last section is used to calculate the loss due to deposition.

The fission products were divided into four classes according to their deposition properties. The four classes are refractories, noble gases, volatiles, and halogens. Refractory fission products were assumed to condense, along with UO_2 , soon after vaporization, hence to be particulate in each containment, including the pressure vessel. Noble gases were assumed to be molecular throughout the containment system. The volatile fission products were assumed to include all other fission products except the halogens. The volatile fission products were assumed to be molecular in the pressure vessel and particulate in each successive containment. Half of the halogens were assumed to remain molecular throughout like the noble gases, the other half was assumed to be molecular in the pressure vessel and particulate in each successive containment, like the volatile fission

products. The typical fission products of the four classes are listed in Table 2.

We can now write an expression for the rate of change in the inventory in the pressure vessel.

$$dP/dt = V_u F' Q/M_u - L_p (P/V_p - C/V_c) - \lambda P - K'_c P^2 - K_{sp} P \quad (14)$$

where the K'_c is different from the K_c in Equation (2) in that K'_c is expressed in the units $\text{cm}^3 \text{curie}^{-1} \text{sec}^{-1}$, while K_c is expressed in the units $\text{cm}^3 \text{sec}^{-1}$. As mentioned above, K'_c and K_{sp} are equal to zero for those fission products which are molecular in the pressure vessel.

In the containment, provision was made to account for removal of particles by filtration or of iodine by charcoal absorption and the loss rate by filtration-adsorption is $(F_c E_c / V_c) C$. Of course, this term is zero for noble gas isotopes. The differential equation for the fission product inventory in the containment can therefore be written:

$$\begin{aligned} dC/dt = L_p (P/V_p - C/V_c) - L_c (C/V_c - B/V_b) \\ - \lambda C - F_c E_c C/V_c - K'_c C^2 - K_{sc} C \end{aligned} \quad (15)$$

If the containment is enclosed by a building and if the reactor building is enclosed with a berm containment (Appendix VII of Auxier and Chester, 1973), then the equation which described the inventory in the reactor building and access tunnel is entirely analogous to those above for the containment.

TABLE 2
FOUR CLASSES OF FISSION PRODUCTS

Class Number	Description	Nuclei
I	Noble Gases	Kr, Xe
II	Halogens	I, Br
III	Volatiles	Ge, Se, Rb, Sr, As, Mo, Tc, Ru, Rh, Pd, Ag, Cd, In, Sn, Sb, Te, Cs, Ba
IV	Refractories	Y, Zr, Nb, La, Ce, Pr, Nd, Pm, Sm

$$\begin{aligned}
 dB/dt = & L_c (C/V_c - B/V_b) - L_b (B/V_b - T/V_t) - \lambda B \\
 & - F_b E_b B/V_b - K_c 'B^2 - K_{sb} B
 \end{aligned} \tag{16}$$

$$\begin{aligned}
 dT/dt = & L_b (B/V_b - T/V_t) - L_t T/V_t - \lambda T \\
 & - F_t E_t T/V_t - K_c 'T^2 - K_{st} T .
 \end{aligned} \tag{17}$$

Consider these cases: an unenclosed containment, a containment enclosed in a reactor building and a containment enclosed in a building which is enclosed in an underground or berm containment so that the building leaks only to an access tunnel which in turn leaks to the environment. All these cases are shown in Figures 1 through 4.

With an unenclosed containment, the release R changes according to

$$dR/dt = (L_c/V_c) C - \lambda R . \tag{18}$$

With a containment enclosed in a building, the differential equation becomes

$$dR/dt = (L_b/V_b) B - \lambda R . \tag{19}$$

If the building is enclosed, then the leak to the environment from the access tunnel of the underground containment becomes

$$dR/dt = (L_t/V_t) T - \lambda R . \tag{20}$$

The three equations are analogous to one another; they pertain respectively to single, double, and triple containment systems.

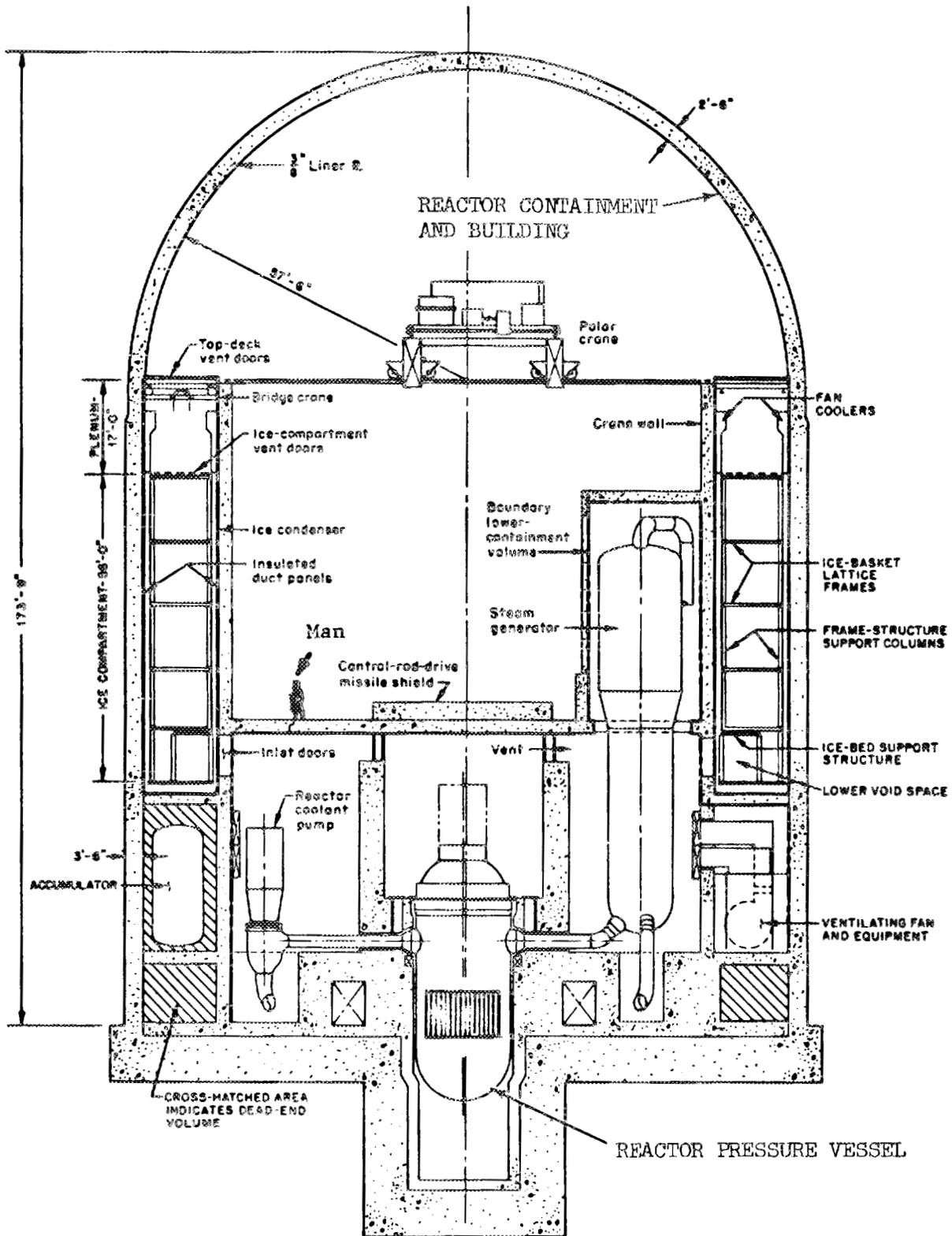


Fig. 1. A Section Through a Typical Primary Coolant System and Containment for a Large PWR System

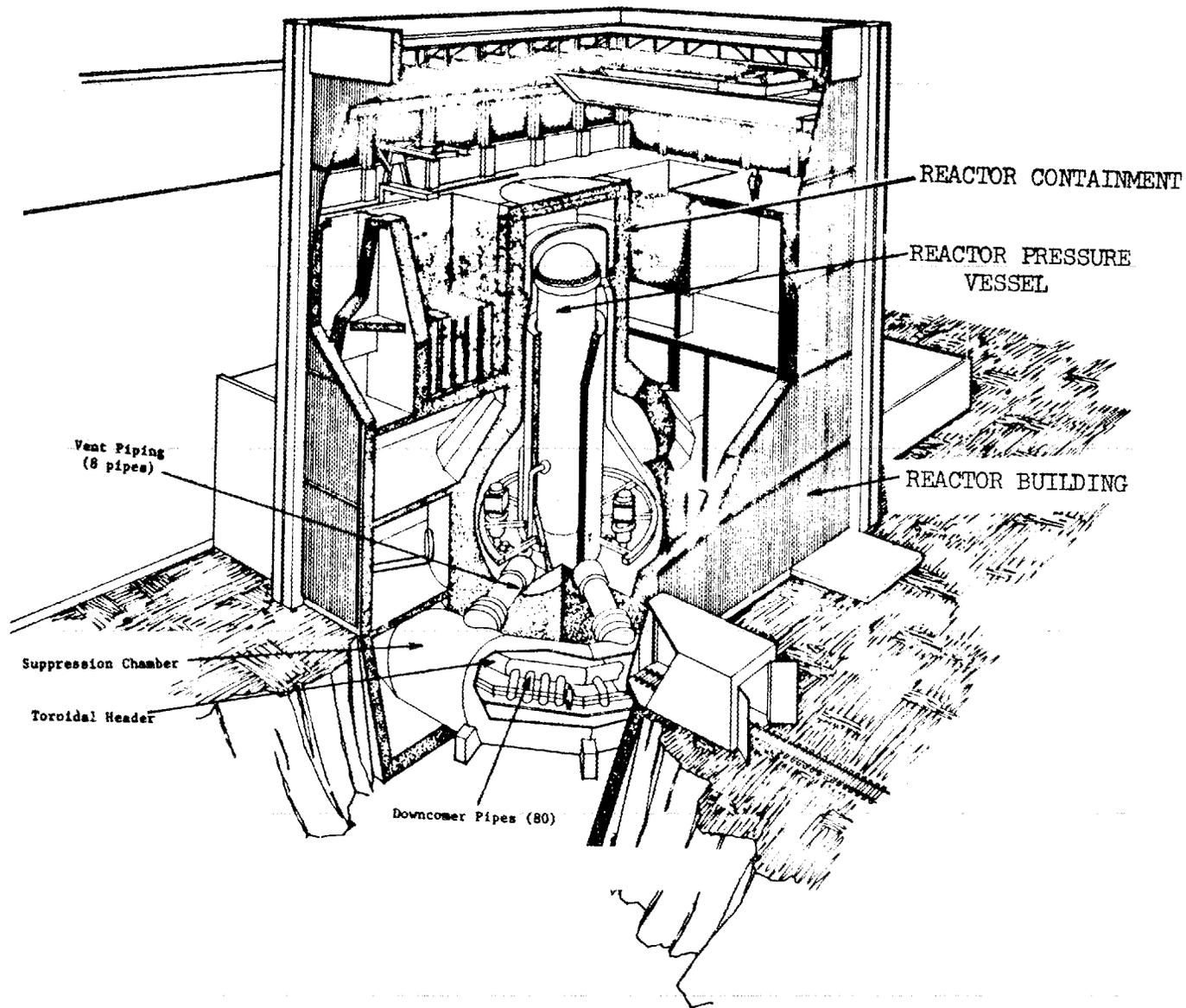


Fig. 2. Cutaway View of a BWR Secondary Containment Building Showing Primary Containment System Enclosed

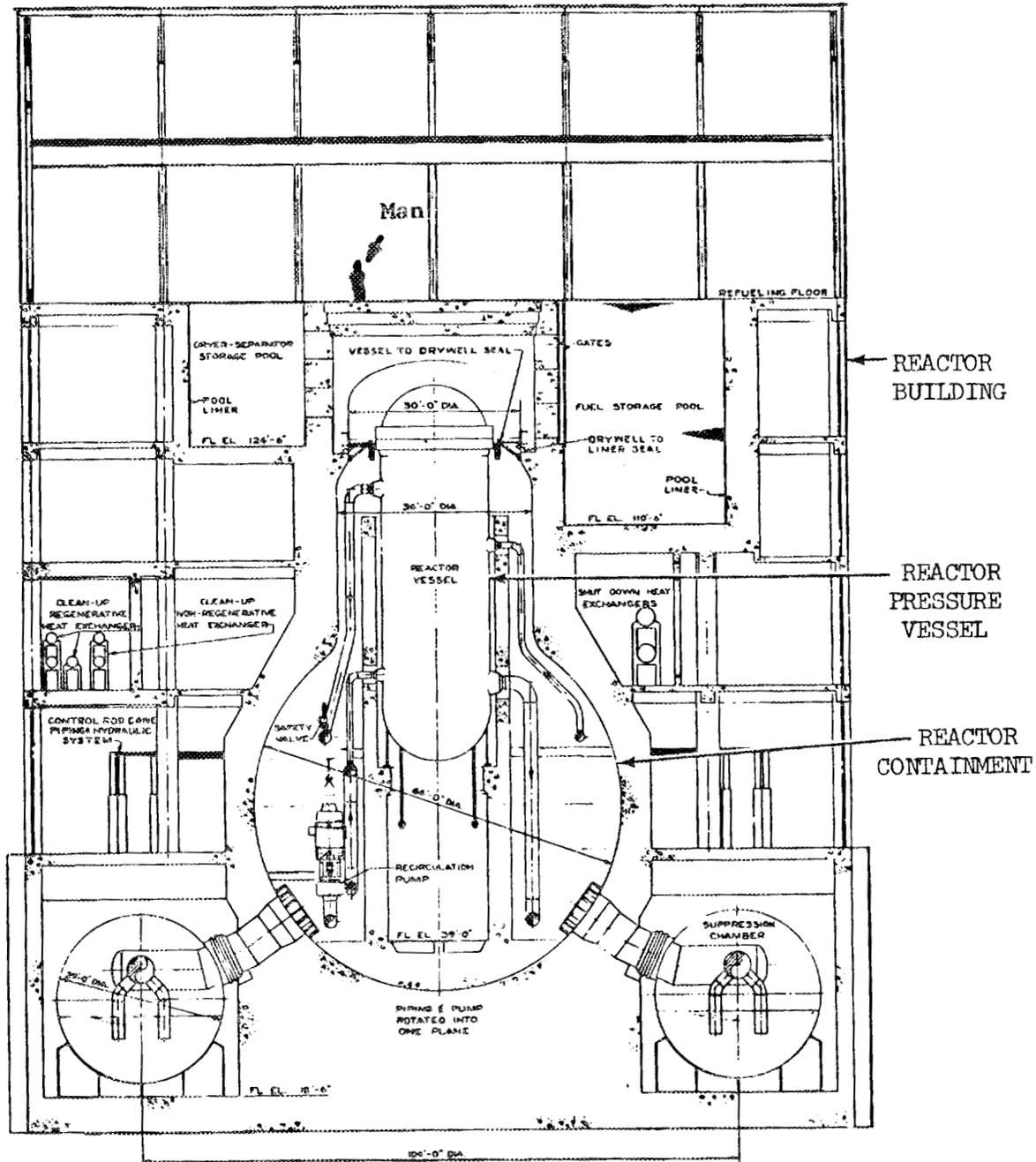


Fig. 3. A Section Through a Typical Primary Coolant System and Containment for a Large BWR System

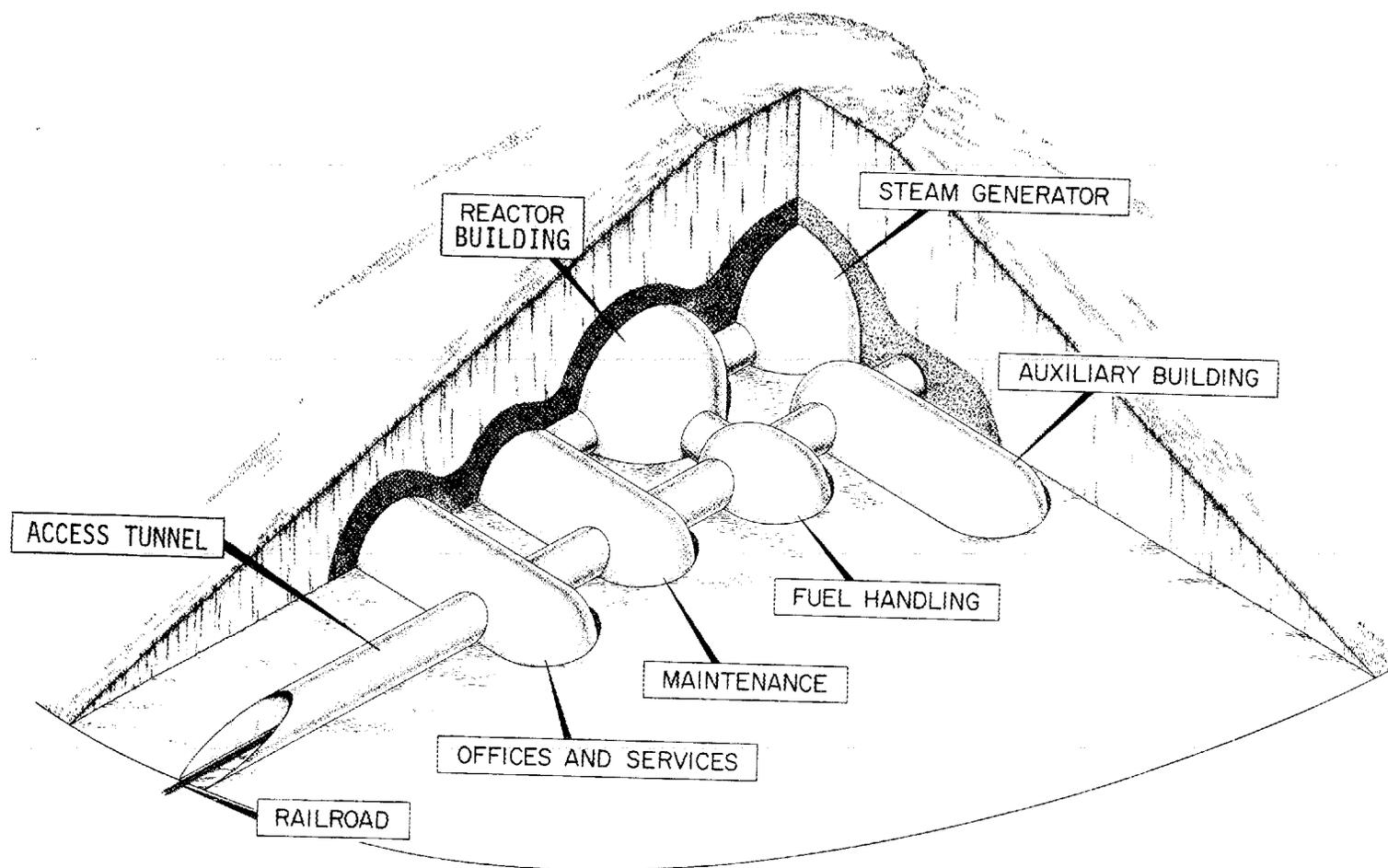


Fig 4. Cutaway View of a Berm Containment for a Large FBR System

Equations (14) through (20) are solved by means of a finite difference technique.

IV. THE RELATIVE EFFECTS OF VARIOUS PARAMETERS ON THE FISSION PRODUCT RELEASE

A damaged BWR (boiling water reactor) was chosen to be the normal case in calculating the relative effects of various parameters on fission product release. The values of parameters used are shown in the Appendix, Table 8, Table 9, and Table 10. In each case, one of the parameters is decreased or increased with respect to the normal case to see its effect on release.

Since the effects of parameter variation are different for the four different classes of fission products, ^{133}Xe , ^{131}I , ^{90}Sr , ^{91}y were chosen to represent the noble gases, halogens, volatiles, and refractories respectively. The effects of various parameters on the releases of these four fission products are shown in Figures 5, 6, 7, and 8 respectively.

Figure 5 shows that varying vaporization rate has small effect on ^{133}Xe release at times shortly after reactor blowdown, and negligible effect at longer times. The effect of a longer blowdown time is to reduce the ^{133}Xe release at short times. But this effect is negligible at longer times. Since noble gases were presumed to be molecular through the containment system, the release is not affected by changing aerosol parameters.

Figure 6, page 22, shows that the effect of a longer blowdown time on the ^{131}I release is the same as that on the ^{133}Xe release. Changing the vaporization rate or using a larger particle fall

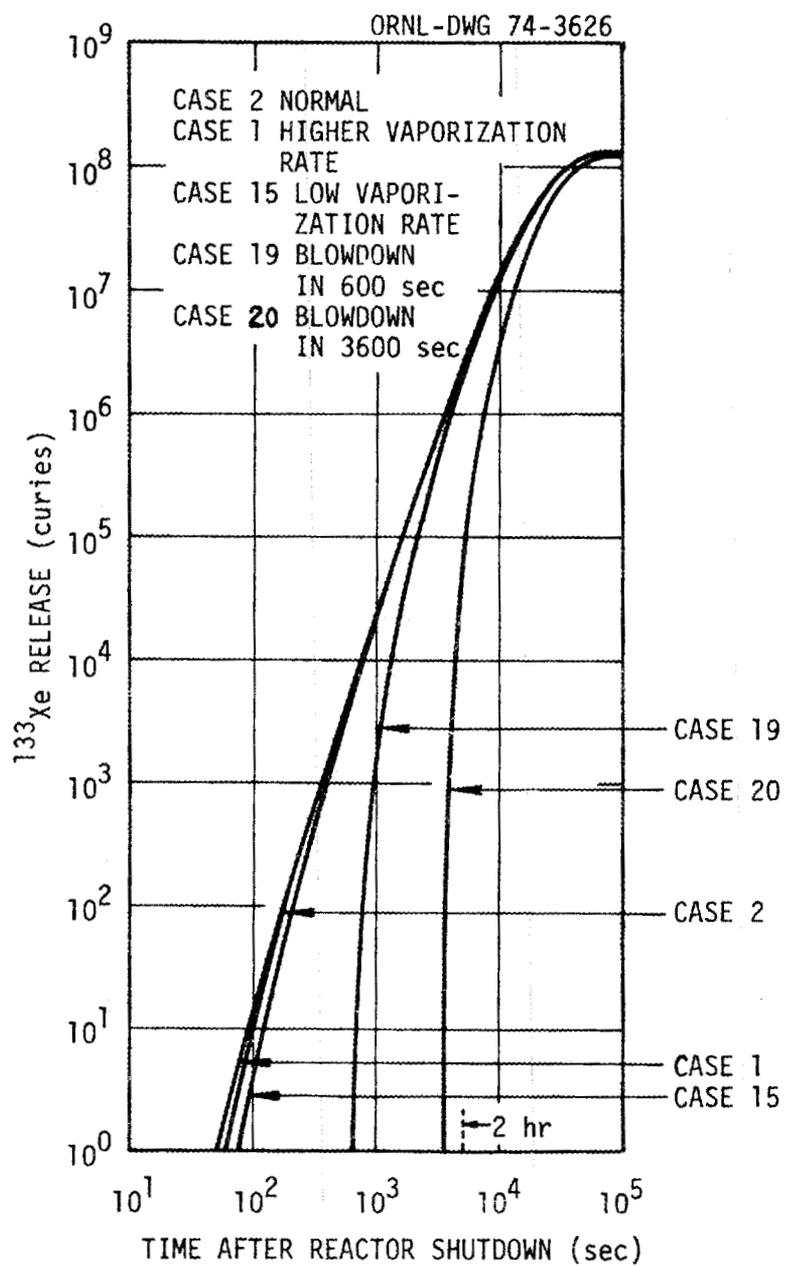


Fig. 5. Effects of Various Parameters on ^{133}Xe Release

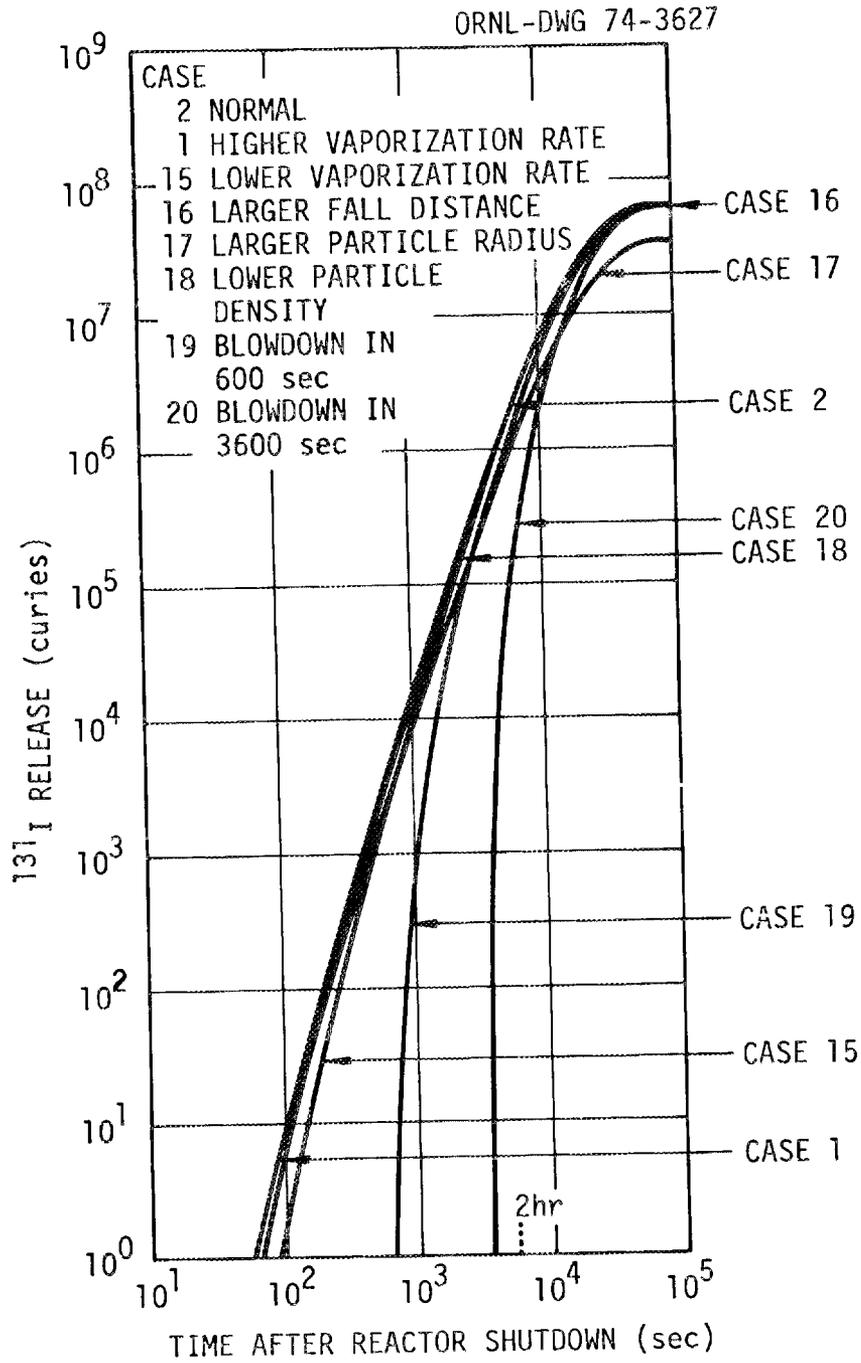
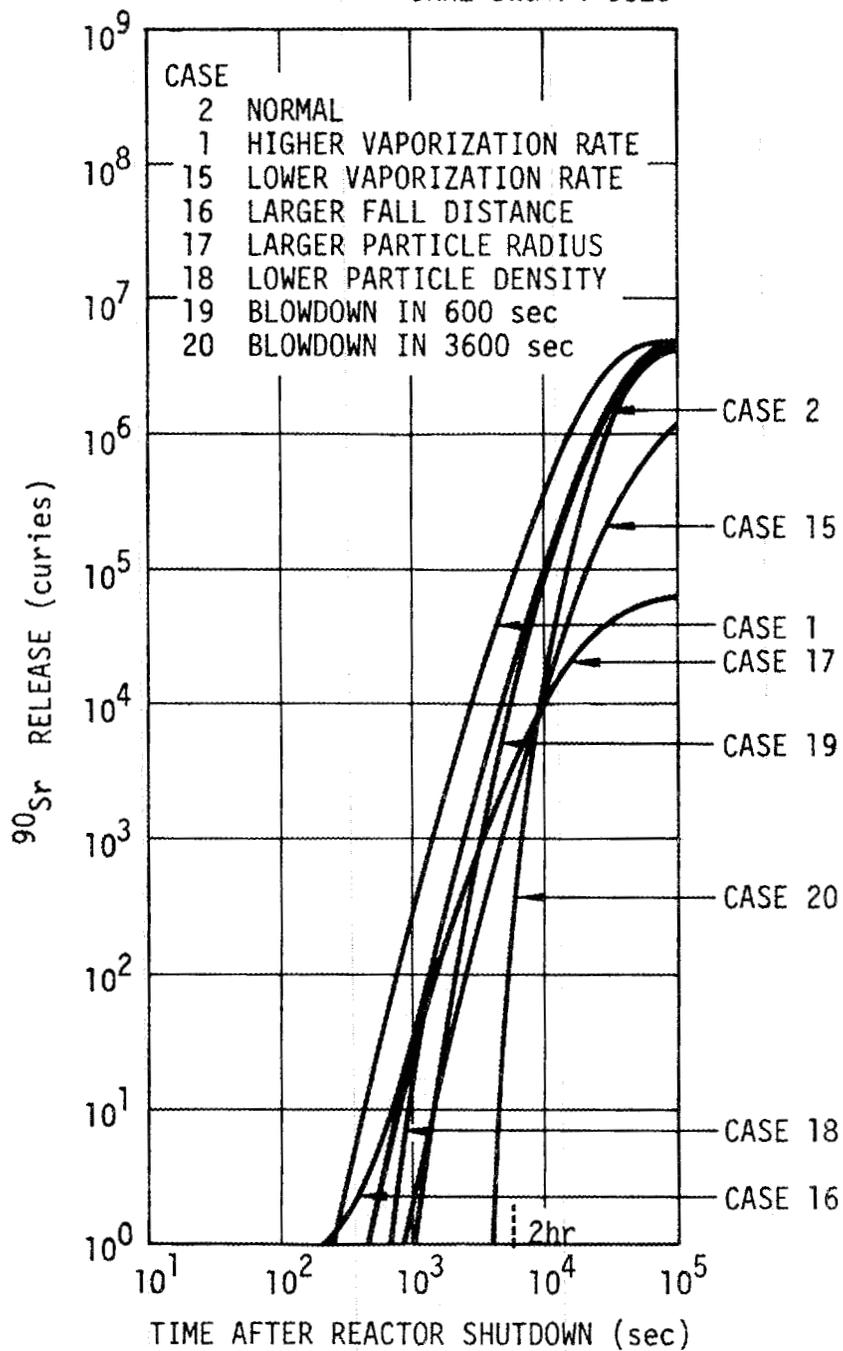
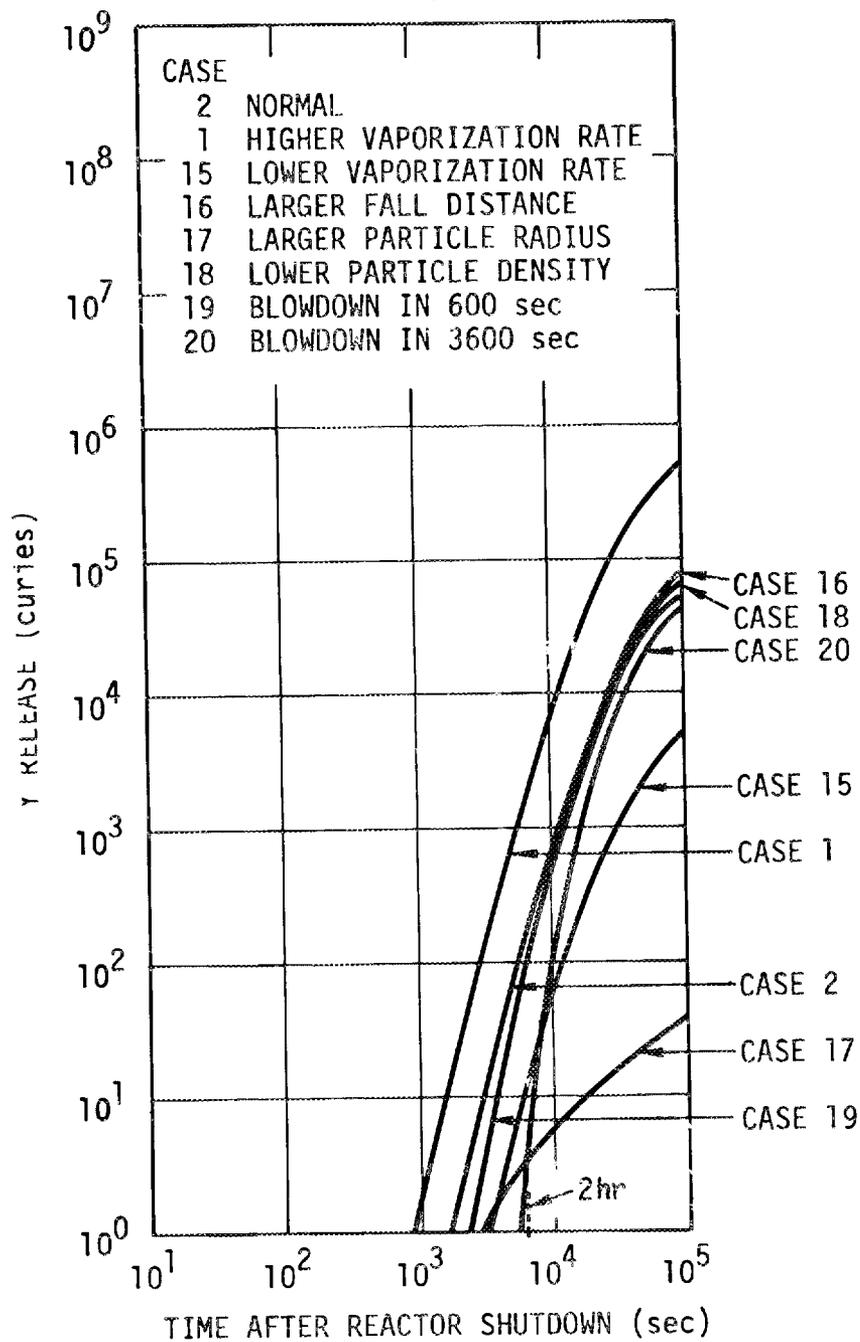


Fig. 6. Effects of Various Parameters on ¹³¹I Release

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Fig. 7. Effects of Various Parameters on ^{90}Sr Release

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Fig. 8. Effects of Various Parameters on ^{91}Y Release

distance or lower particle density has no significant effect on the ^{131}I release. But increasing the minimum particle radius leads to a significant reduction of the ^{131}I release at longer times.

Figure 7, page 23, shows that the effect of increasing the minimum particle radius on the ^{90}Sr release is very large. Increasing the UO_2 vaporization rate leads to an increase of the ^{90}Sr release. Decreasing the UO_2 vaporization rate changed the magnitude of the ^{90}Sr release more than increasing the UO_2 vaporization rate did. This is also true for ^{131}I release. Although particle fall distance does not appear to be an important parameter, it leads to a slight increase of the ^{90}Sr release at short times. Longer blowdown time leads to a reduction of ^{90}Sr release at times shortly after blowdown but this effect is less important for the ^{90}Sr release than for the ^{131}I and ^{133}Xe release.

Figure 8 shows that the effects of changing the vaporization rate and changing the aerosol parameters are more significant for ^{91}Y release than for the other three fission products. On the other hand, the effect of a longer blowdown time is relatively unimportant for ^{91}Y release.

In summary, the minimum particle radius is the most important parameter in determining the magnitude of the release of particulate fission products. For refractories an increase of the minimum particle radius by a factor of ten can lead to a three orders of magnitude reduction of release at the end of 10^5 sec after reactor shutdown.

The assumed fraction of decay heat available to vaporize UO_2 , which determines the vaporization rate, is important for fission products with vapor pressure equal or lower than that of ^{90}Sr but is not important for fission products with higher vapor pressure. The reason is that high vapor pressure fission products were released from the core much faster than they were leaked through the containments; hence, the release rate was not controlled by the vaporization rate. This reasoning also explains why decreasing the vaporization rate had more effect on the ^{90}Sr release than on the ^{131}I release.

The various parameter changes and effects are briefly summarized in Table 3.

V. CONTAINMENT EFFECTS

Our calculation shows that the effects of containment systems on the four different classes of fission product releases are similar. Hence only the effect on the ^{91}Y release is shown here.

Figure 9 shows that the addition of even a damaged access tunnel reduces the release by two orders of magnitude. And the fission product release of a BWR with damaged reactor building and damaged access tunnel is about the same as that of a BWR with design basis reactor building but with no access tunnel.

The effect of containment filter on the ^{91}Y release of a PWR is also shown in Figure 9. The calculation shows that the filter can lead to a reduction of the release to the environment by two orders of magnitude. In other words, the effect of adding a containment filter is about the same as replacing a damaged access tunnel by a design basis access tunnel.

TABLE 3
VARIOUS PARAMETER CHANGES AND EFFECTS

Parameter Change	Effect on Magnitude of Release (at 10^5 sec)
Higher Vaporization Rate	Increases
Greater Fall Distance	Increases Slightly
Greater Minimum Particle Radius	Decreases Greatly
Lower Particle Density	Increases Slightly
Blowdown in 600 sec	Remains Unchanged
Blowdown in 3600 sec	Remains Unchanged

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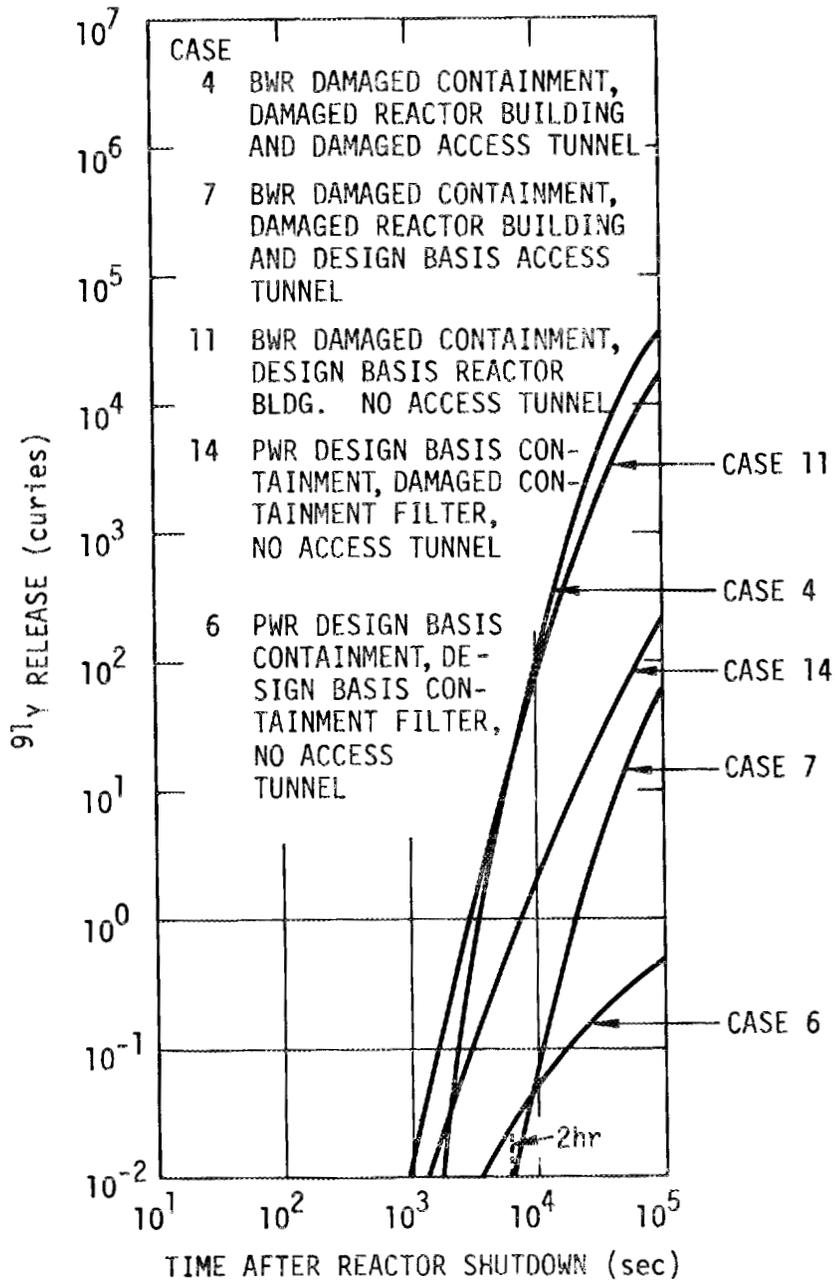


Fig. 9. Effects of Various Containment Systems on ⁹¹Y Release

CHAPTER III

THE EFFECT OF THE FISSION-PRODUCT VAPORIZATION

ON DECAY HEAT AVAILABILITY

I. STAGES OF CALCULATION

At the first stage of our calculation, the fission products were divided into four classes: noble gases, halogens, volatiles, and refractories. And it was assumed that the fractions of each fission product remaining in the core as a function of time are the same for all fission products in the same classification, and ^{133}Xe , ^{131}I , ^{90}Sr , ^{91}Y were selected to represent the four classes of fission products, respectively.

First, $H'(t)$ in Equations (7) and (8) was set equal to one. The fraction of ^{133}Xe remaining in the core as a function of time was obtained by using Equations (5) through (13) with the aid of the computer. Then this fraction was multiplied by the fraction of the total decay heat attributable to all noble gases. The latter fraction is also a function of time. The same calculation was also done for ^{90}Sr , ^{131}I , and ^{91}Y .

Then, the results for the four fission product classifications were summed. The sum was the first approximation to the fraction of decay heat remaining in the core and was to be used as the value of $H'(t)$ in the calculation of the second approximation.

Because this effect, as mentioned before, is a negative feedback effect, and also because the result obtained in the first approximation is quite different from unity, this procedure was

iterated for ten runs to get the final convergent result. The final convergent value is defined as the decay heat "correction factor".

Since the vapor pressure of different fission products in the same classification varied over several orders of magnitude (Bedford and Jackson, 1965), it was decided that the second stage was to classify the fission products into more than four classes according to their vapor pressure. The reclassification of fission products into ten groups is shown in Table 4.

In Table 4, it is worth noting that although the theoretical value of Te vapor pressure is high, a lower value from experimental work (Parker, et al., 1967) is used in our classification since Te can be dissolved in Zr cladding (Appendix VI of Auxier and Chester, 1973). The results obtained by using ten fission product groups rather than four will be shown and discussed in the next section.

The decay heat available for vaporizing fission products is reduced appreciably. But the most important thing after the accident is the fission product release rather than the decay heat itself. Hence, the final stage was to use the modified decay heat to calculate the fission product release. The results are compared with some available data. The comparison will be given in the last section of this chapter.

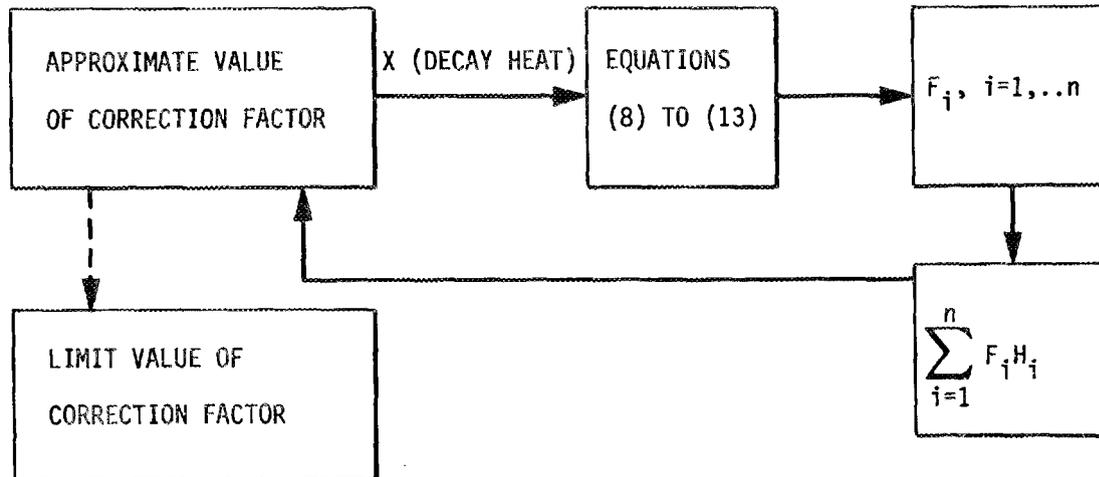
The three stages of calculation described above are illustrated by Figures 10 and 11.

There is one thing that should be pointed out here. Although ten groups of fission products were used in calculating the fraction of decay heat remaining in the core, the original four classifications

TABLE 4
 FISSION PRODUCT GROUPS WITH THE RATIO OF THEIR
 VAPOR PRESSURE TO THAT OF UO_2

Group Number	Nuclei	Ratio of their vapor pressure to that of UO_2
1	As, Se, Br, Kr, Rb, Cd, I, Xe, Cs	1.0×10^6
2	Ge, Sr, Pd, Sn	8.0×10^2
3	Ag, In	1.0×10^4
4	Sb, Ba	1.5×10^5
5	La, Nd, Pm	70.0
6	Ge, Pr, Te	8.0
7	Rh, Sm	1.0
8	Y, Nb	0.25
9	Zr, Ru	3.0×10^{-2}
10	Mo, Tc	7.0×10^{-3}

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where:

- F_i = fraction of class i fission products not vaporized
- H_i = fraction of decay heat contributed by class i fission products
- n = 4 in the first stage of calculation
- n = 10 in the second stage of calculation

Fig. 10. The First and Second Stages of Calculation of Effect of Fission Product Loss on Decay Heat

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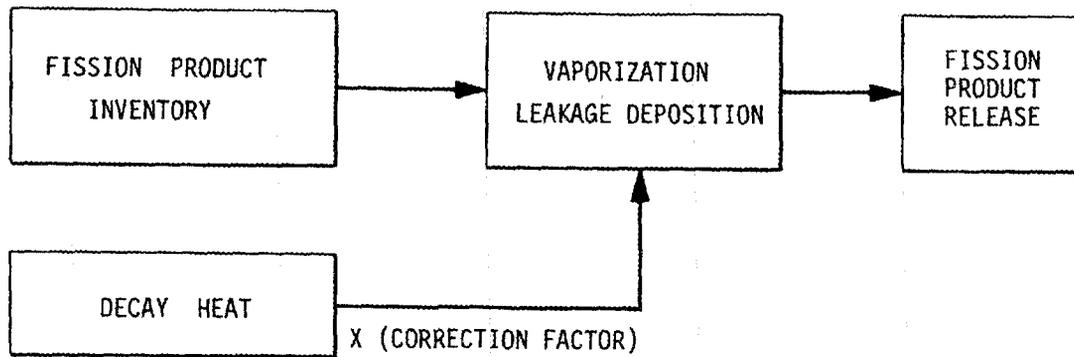


Fig. 11. The Third Stage Calculation of Effect of Fission Product Loss on Decay Heat

were used to calculate the fission product release. This is due to the fact that the original classifications were based on the rate of deposition of the fission products. In other words, in calculating the decay heat escape fraction the fission products were classified into ten groups according to their vapor pressures, but in calculating the fission product release the fission products were classified into four classes according to their deposition properties.

II. RESULTS AND INTERPRETATION

The decay heat correction factors obtained in stage one and stage two calculations with an H value of 10^{-3} are shown in Figure 12. The higher correction factor of curve A (in Figure 12) shortly after blowdown, the monotonic decrease of curve A (in Figure 12) after 10^3 seconds, and the continued increase of curve B (in Figure 12) after 100 seconds are explained by the following reasoning.

The decay heat fractions contributed by the four and ten classes of fission products are shown in Figure 13 and Table 5, respectively, which are based on the work of R. O. Chester (1973). From Figure 13, it is clear that shortly after blowdown the volatiles contribute to most of the decay heat. By reexamining the vapor pressures of volatiles, it is found that using Sr to represent the volatiles makes the vapor pressure lower than the actual value. This reduces the fission product vaporization, which, in turn, leads to a higher correction factor.

The monotonic decrease with time of curve A (in Figure 12, page 35) after 10 seconds is also due to using Sr to represent the entire group of volatiles. Before 10^3 seconds most of the volatiles escaped

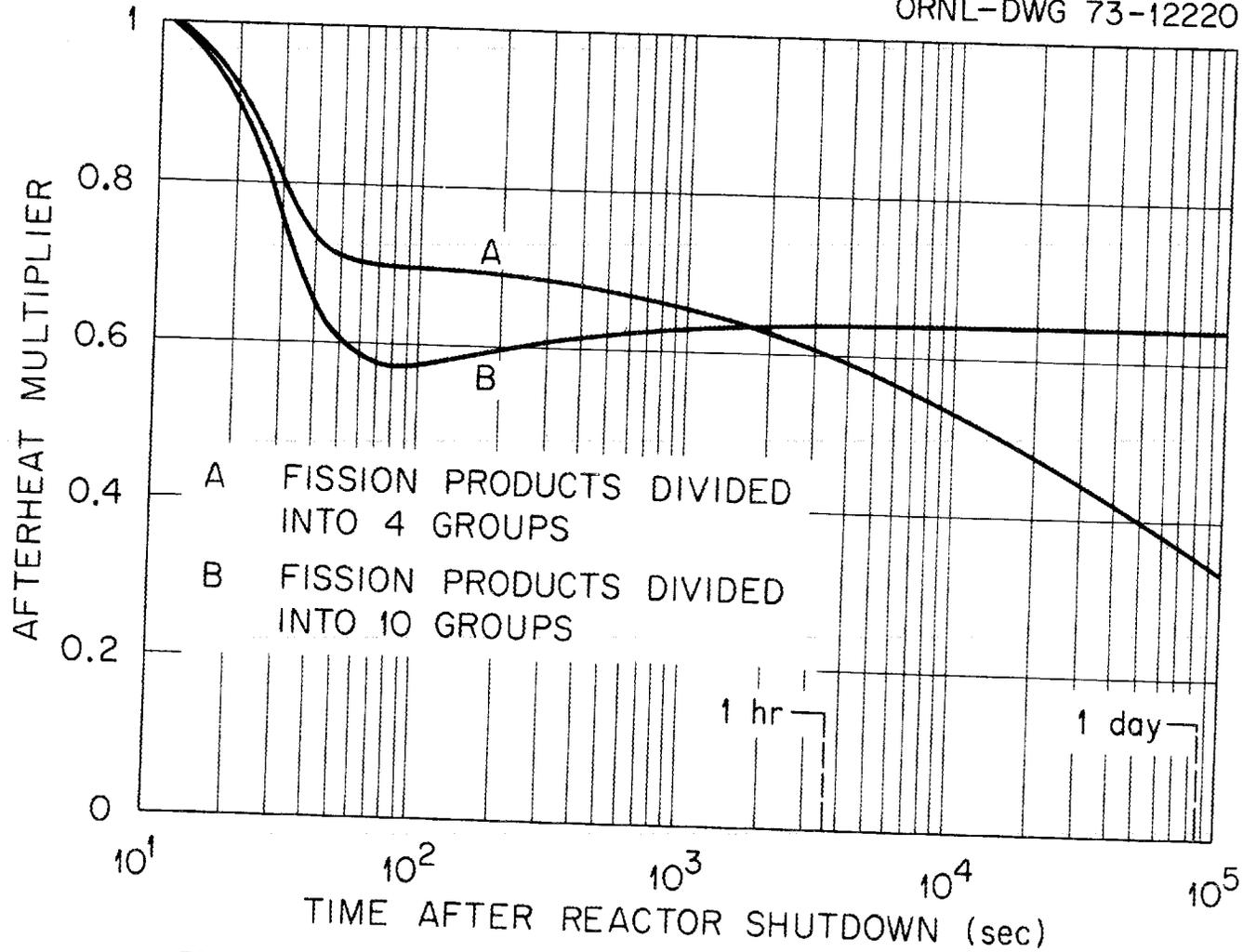


Fig. 12. Calculated Correction Factor Due to Volatized Fission Products for Reactor Core Afterheat

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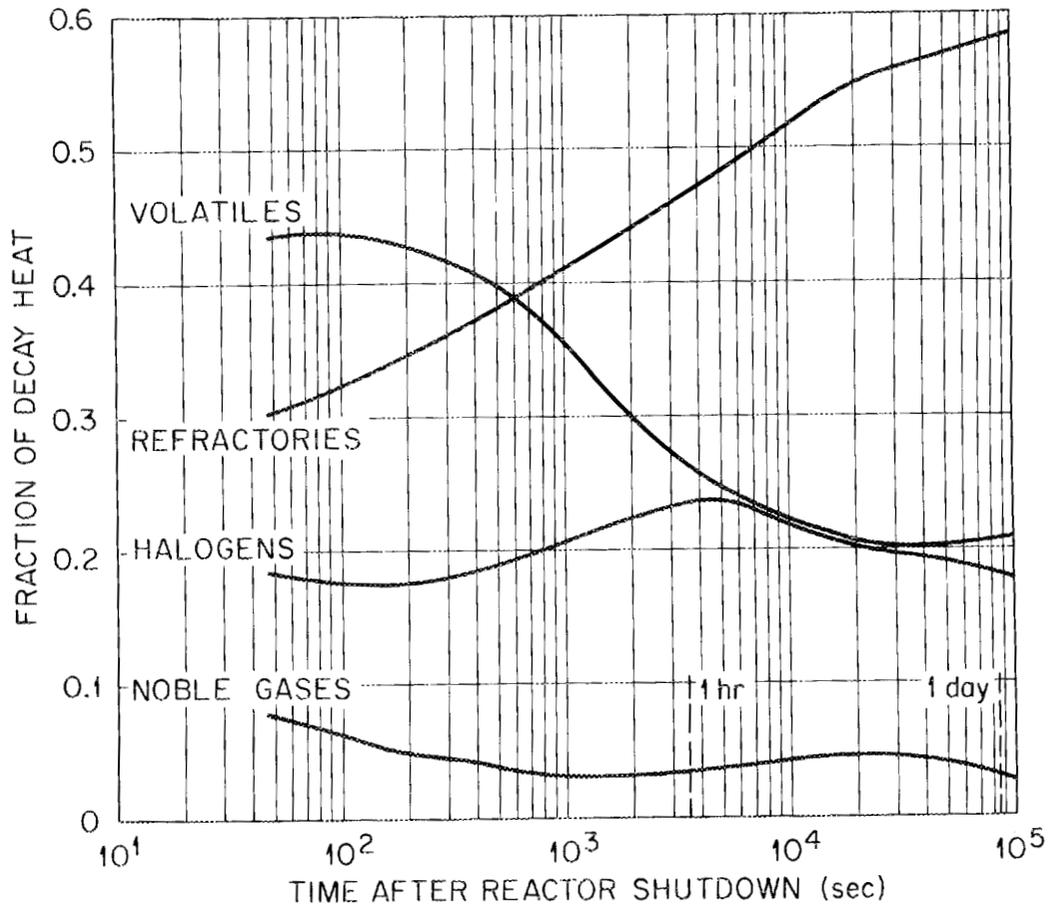


Fig. 13. Decay Heat Contribution Due to Fission Products in Each Classification

TABLE 5
DECAY HEAT FRACTIONS ATTRIBUTABLE TO THE
TEN FISSION PRODUCT GROUPS

Group Number	Fractions at 100 Seconds	Fractions at 10^5 Seconds
1	0.356	0.214
2	0.061	0.050
3	0.000	0.000
4	0.067	0.041
5	0.152	0.197
6	0.145	0.172
7	0.003	0.008
8	0.097	0.169
9	0.040	0.110
10	0.079	0.038

from the core due to the high vapor pressure. But the fraction of Sr which escaped from the core was only 0.167 at 10^3 seconds and 0.995 at 10^5 seconds.

Although at 10^5 seconds the refractories contributed approximately 60 percent of the decay heat (by Figure 13, page 36), they do not contribute too much to the change of correction factor since only a very small fraction of the refractories have vaporized. On the other hand, at 10^5 seconds the volatiles still contribute approximately 20 percent of the decay heat (by Figure 13); and since most of Sr vaporized between 10^3 seconds and 10^5 seconds, the escape of Sr was the primary contributor of the decreasing of correction factor after 10^3 seconds. This fact leads to the decrease of curve A (in Figure 12). The continued increase of curve B (in Figure 13) was due to the smaller escape fraction of refractories.

As mentioned in the third section of Chapter II, the decay heat correction factor also depends on the value assumed for H in Equations (7) and (8). The decay heat correction factors obtained by setting H equal to 10^{-4} , 10^{-3} , 10^{-2} are shown in Figure 14. From Figure 14 it is clear that the correction factor is not sensitive to the change in the value of H, since the correction factor changes by less than a factor of two while H changes by two orders of magnitude.

The effect of fission product escape on the UO_2 vaporization is shown in Figure 15. From Figure 15, the escape of fission products can lead to a 35 percent reduction in the UO_2 vaporization after 10^3 seconds.

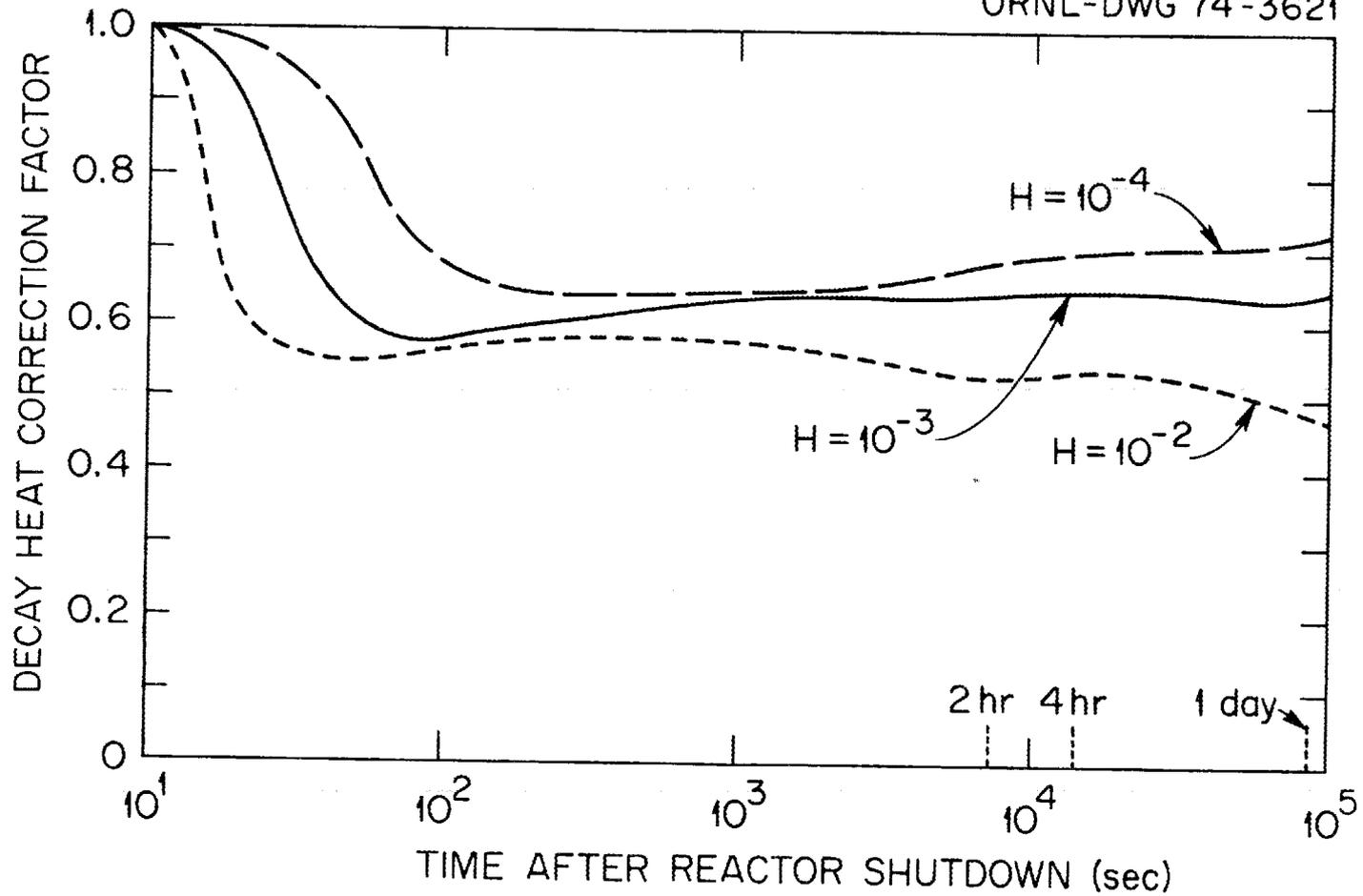


Fig. 14. Decay Heat Correction Factor for Three Different H Values

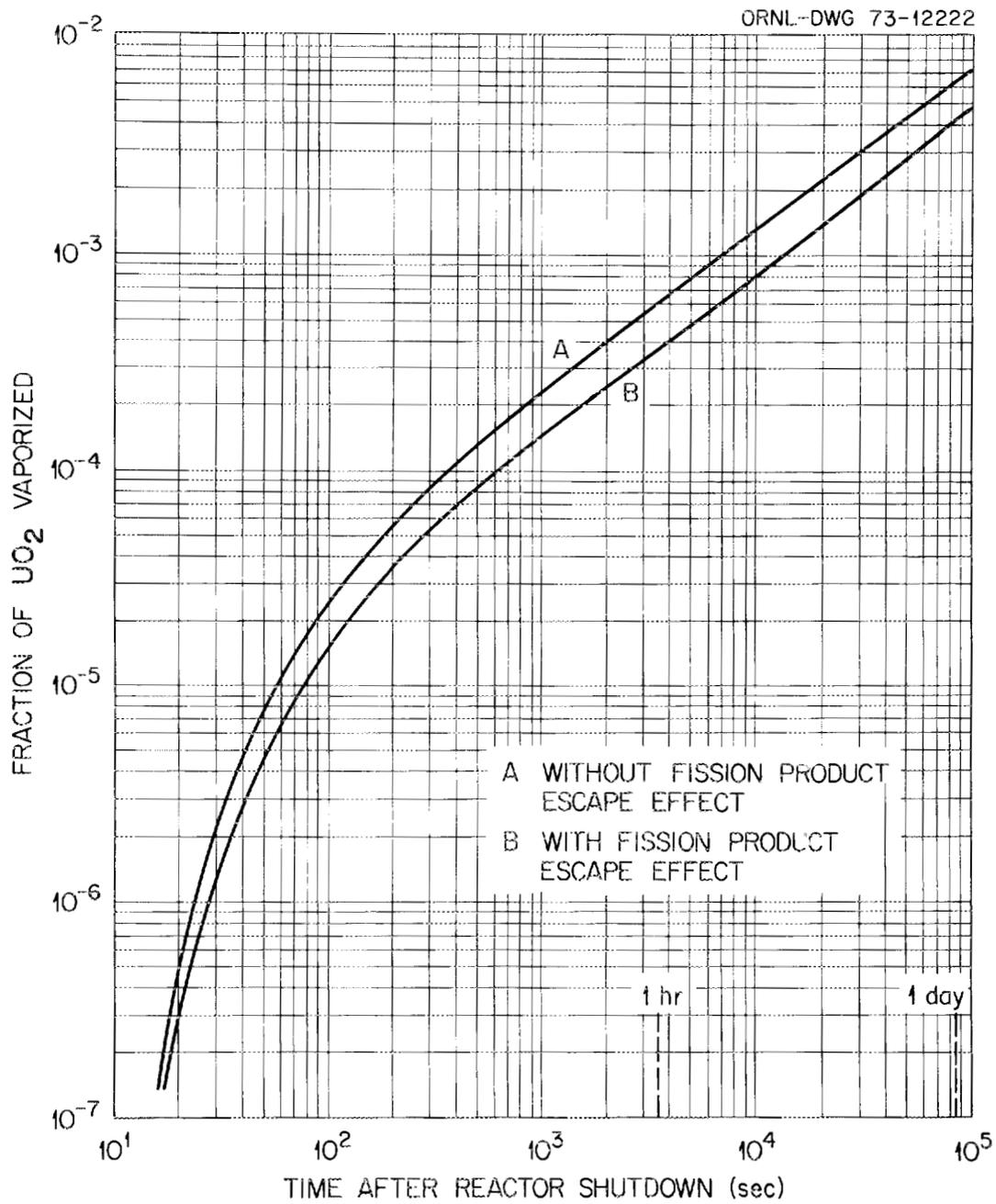


Fig. 15. Fraction of UO_2 Vaporized Under Two Different Conditions

The calculated releases of various fission products for the case including the fission product vaporization effect are compared with those which were obtained without including this effect. This comparison is shown in Table 6. Table 6 shows that fission product vaporization has negligible effect on the release of high vapor pressure fission products, but it can lead to appreciable reduction of the low vapor pressure fission product release.

III. COMPARISON OF THE CALCULATED VALUES WITH THE AEC SUGGESTED VALUES

The AEC regulatory staff (DiNunno et al., 1962) suggests typical values of the amount of radioactivity which leaves a PWR reactor core and reaches the reactor building. The AEC values are essentially the total airborne release from the core. They include natural deposition processes but not the effects of washdown or filtering from protective safeguards. Next, leakage to the environment is assumed to occur at a constant rate of 0.1 percent per day.

In order to compare the calculated values with the AEC suggested values, three numerical values must be examined: First, the fission product inventories in pressure vessel, containment, reactor building and the releases into the environment are summed. Doing the summation in this way, the deposition process was included but the effect of leakage was excluded. The summed values can be compared directly with the AEC suggested values. Parker (1967) suggested values for Cs and Ru based on his experimental work. These values are also included in the comparison which is shown in Table 7. Table 7 shows that the AEC suggested values compared to the calculated values of Zr and Nb

TABLE 6
 FISSION PRODUCT RELEASE AT 10^5 SECONDS AFTER A DESIGN BASIS
 ACCIDENT (WITH AND WITHOUT FISSION PRODUCT ESCAPE EFFECT)

Nuclei	Without FPEE (Ci)	With FPEE (Ci)
Xe-133	0.21×10^6	0.21×10^6
I-131	0.87×10^5	0.87×10^5
Sr-90	0.44×10^4	0.38×10^4
Y-91	0.54×10^2	0.34×10^2
Nb-95	0.60×10^2	0.38×10^2
Zr-95	0.72×10^1	0.46×10^1
Ru-103	0.79×10^1	0.50×10^1
Ru-106	0.27×10^1	0.16×10^1
Te-129m	0.23×10^3	0.15×10^3
Cs-137	0.69×10^4	0.69×10^4
Ba-140	0.10×10^6	0.10×10^6

TABLE 7
 COMPARISON OF THE CALCULATED FISSION PRODUCT SOURCE
 TERM WITH SUGGESTED VALUES

Fission Product	Suggested Value		Sum	Calculated Value ^{††}	
	Percent of Inventory in Bldg.	to Envir*		Percent of Inventory in Bldg	to Envir
Xe	100*	.012	47	30	.01
I	50*	.003	46 [†]	28 [†]	.0098 [†]
Zr	1*	1.2×10^{-4}	2×10^{-3}	9×10^{-4}	2×10^{-7}
Nb	1*	1.2×10^{-4}	2×10^{-2}	7×10^{-3}	2×10^{-6}
Ba	1*	1.2×10^{-4}	36	22	7×10^{-3}
Sr	1*	1.2×10^{-4}	36	15	4×10^{-3}
Cs	50**	.006	3.8	2.3	8×10^{-4}
Ru	5**	6×10^{-4}	2×10^{-3}	1×10^{-3}	2×10^{-7}
Gross Activity	15*	1.8×10^{-3}	16	10	3.5×10^{-3}

*These values are suggested by AEC in TID-14844 (1962).

**These values are suggested by Parker, et al. (1967).

[†]In this particular calculation, 50 percent of the core inventory of iodine is considered to have aerosol properties similar to the rare gases. As such, these numbers may be an overestimate of the release.

^{††}The calculated values are obtained at 10^4 sec, and assume 10^{-3} of the decay heat available for the evaporation of UO_2 . Radiological decay of all isotopes is included in these values.

overestimated the release by at least a factor of 50, but the values for Ba and Sr were underestimated by a factor of 36. Iodine was slightly overestimated. Xenon was overestimated by a factor of two. The higher calculated values of Sr and Ba releases can be explained by the following reasoning: because of the zirconium-steam reaction, the atmosphere surrounding the fuel changed from oxidizing condition to reducing condition. Hence, the vapor pressure of Sr and Ba increased by more than two orders of magnitude (Bedford and Jackson, 1965). Further, it should be noted that the calculated values are a nonlinear function of time. The time chosen for comparison is 10^4 seconds after shutdown. This is approximately three hours, the longest time realistically estimated before back-up emergency assistance will have arrived at the reactor site (Auxier and Chester, 1973). Also, in many circumstances in three hours the pressure vessel will not yet have melted through requiring a basic geometry change in the calculation. Second, the calculated airborne inventory in the reactor building at 10^4 seconds is listed in Table 7. Third, the calculated total release to the environment at 10^4 seconds is listed.

There is another interesting thing which should be pointed out here. By intuition, one might guess that the release source term of high vapor pressure fission products is greater than that of low vapor pressure fission products. But the values of Cs, Ba, and Sr in Table 7 show that the intuition is wrong. This is because a higher concentration produces a more than compensating higher deposition rate.

CHAPTER IV
ESTIMATION OF THE MAGNITUDE OF THE EFFECTS
OF METAL-STEAM REACTION

If the heat released from the zirconium-steam reaction as a function of time is to be determined very accurately, the detailed mechanical and geometrical changes following the reactor accident are needed. Since the objectives of this study were to investigate the effect of zirconium-steam reaction on the fission product release rather than the reaction itself and the general characteristics of this effect rather than that of a single and unique accident, an assumption was used to avoid this complexity which is based on the results obtained by Louis Baker and R. O. Ivins (1965).

It was assumed that the zirconium-steam reaction starts at 40 sec after the reactor shutdown and 20 percent of the total zirconium reacts within the first half hour at a constant rate. This constant rate is determined by the following calculations.

The zirconium-steam reaction heat at 1800 °K is 137,780 cal/mole or 6.359×10^3 joules/g (Lemmon, et al., 1959). And the total mass of zirconium is 137.172 pounds or 0.6×10^8 g (see, for example, USAEC Docket No. 50-259/260/296, 1973). Hence

$$\begin{aligned} \text{Total reaction heat} &= 6.359 \times 10^3 \text{ joules/g} \times 0.6 \times 10^8 \text{ g} \\ &= 3.815 \times 10^{11} \text{ joules} \end{aligned}$$

The assumed constant reaction heat rate

$$= 3.815 \times 10^{11} \text{ joules} \times 0.2 \div 1800 \text{ sec}$$

$$= 4.24 \times 10^7 \text{ joules/sec .}$$

Since the decay heat is distributed within the fuel rod where the fission products are stored while the zirconium-steam reaction heat is distributed outside the fuel rod, the fraction of zirconium-steam reaction heat available for the vaporization of the fission products, which is denoted by H_1 in the following calculations, should be less than that of the decay heat. And if there is some cooling water left in the core, the water should be in contact with zirconium-cladding rather than the inside of the fuel rod. Hence the fraction of heat used to vaporize the water should be greater for zirconium-steam reaction heat than for decay heat. Both effects lead to the conclusion that H_1 should be less than H . Hence in our calculation, H_1 was assumed to be one order of magnitude less than H .

Incorporating the zirconium-water reaction to the decay heat by combining Equations (7), (8), and (21), the following equations are obtained:

$$P'(t) = 0.140 \times 60^{-0.291} H H'(t) P_0(t-10)/50$$

$$\text{for } 10 \leq t < 40 \quad (22)$$

$$P'(t) = 0.140 \times 60^{-0.291} H H'(t) P_o(t-10)/50 + 4.24 \times 10^7 H_1$$

$$\text{for } 40 \leq t < 60 \quad (23)$$

$$P'(t) = 0.140 H H'(t) P_o t^{-0.291} + 4.24 \times 10^7 H_1$$

$$\text{for } 60 \leq t < 2200 \quad (24)$$

$$P'(t) = 0.140 H H'(t) P_o t^{-0.291}$$

$$\text{for } t \geq 2200 . \quad (25)$$

Inserting Equations (22), (23), and (24) into Equations (9) and (10),

$$-dM_u/dt = A 60^{-0.291} (t - 10)/50 \quad \text{for } 10 \leq t < 40$$

$$= A 60^{-0.291} (t - 10)/50 + B \quad \text{for } 40 \leq t < 60$$

$$= A t^{-0.291} + B \quad \text{for } 60 \leq t < 2200$$

$$= A t^{-0.291} \quad \text{for } t \geq 2200$$

where

$$A = 6.511 \times 10^{-5} H H'(t) P_o \quad (26)$$

$$B = 1.971 \times 10^4 H_1 .$$

In order to compare the importance of the zirconium-steam reaction heat to that of the decay heat, the first and second terms in the right side of Equation (26) were calculated separately. In this calculation, P_o is set equal to 3.44×10^9 watts and H is set equal to 10^{-3} . From Figure 10, $H'(60)$ equals 0.588 and $H'(1800)$ equals 0.564.

At t = 60 seconds

$$\begin{aligned} \text{the first term} &= 6.511 \times 10^{-5} \times (60)^{-0.291} \times 10^{-3} \\ &\times 0.588 \times 3.44 \times 10^9 = 4.001 \times 10^1 \text{ g/sec} \end{aligned}$$

$$\begin{aligned} \text{the second term} &= 1.971 \times 10^4 \times 10^{-4} \\ &= 1.971 \text{ g/sec} \end{aligned}$$

At t = 1800 seconds

$$\begin{aligned} \text{the first term} &= 6.511 \times 10^{-5} \times (1800)^{-0.291} \times 10^{-3} \\ &\times 0.564 \times 3.44 \times 10^9 = 1.426 \times 10^1 \text{ g/sec} \end{aligned}$$

$$\begin{aligned} \text{the second term} &= 1.971 \times 10^4 \times 10^{-4} \\ &= 1.971 \text{ g/sec} \end{aligned}$$

The ratio of the second term to the first term equals 4.93 percent at 60 seconds and 13.82 percent at 1800 seconds.

From the results of the above calculation, it is evident that while the contribution of zirconium-steam reaction heat to the vaporization of fission products is significant, it is not as large as the contribution from decay heat.

CHAPTER V

SUMMARY

The reactor accident in this study was assumed to be a double-ended break of a large primary-system water pipe and concurrent failure of the emergency core cooling system. Following the reactor blowdown, the decay heat raised the core temperature. A certain fraction of the decay heat was assumed to vaporize UO_2 . The vaporization of various fission products was assumed to be controlled by their vapor pressure relative to that of UO_2 . The airborne fission products were then subjected to decay, deposition and leakage processes, and finally released to the environment.

In this study the transport of fission products after release was not treated. Only agglomeration and gravitational settling were considered in the airborne fission product deposition process. The pressure vessel melt-through which may occur as soon as 2 hours after pipe rupture was not considered.

The assumption of a constant fraction of decay heat available to vaporize UO_2 was investigated in this study by taking into account the decay heat loss due to vaporization of fission products and by changing the assumed fraction.

The effect on fission product release of adding the metal-water reaction heat to the heat source was investigated.

The effect of multiple containments, the effect of a containment filter, and the effect of various parameters (particle fall distance, minimum particle radius and particle density) in the aerosol model on fission product release were studied also.

The results showed that the vaporization of fission products would lead to a 20 percent to 50 percent reduction (depending on the value of other parameters) in the decay heat available to vaporize UO_2 .

The release of low vapor pressure fission products is approximately proportional to the assumed fraction of decay heat available to vaporize UO_2 . But the release of fission products with vapor pressure higher than that of Sr is almost independent of the assumed fraction.

The metal-water reaction heat available to vaporize UO_2 is about one order of magnitude less than that of decay heat. Its effect on the fission product release is small. But the zirconium-steam reaction changed the atmosphere surrounding the fuel from an oxidizing condition to a reducing condition, thus changing the vapor pressure of various fission products. This can lead to an increase of Sr and Ba releases by a factor of 30 or more.

The addition of even a damaged access tunnel can reduce the fission product release by more than two orders of magnitude. The containment filter can lead to a reduction of the refractories' releases by more than one order of magnitude at the end of 2 hours after reactor shutdown.

The minimum particle radius is the most important parameter in the aerosol model. For refractory fission products, increasing the minimum particle radius by a factor of ten can lead to a reduction of release by two orders of magnitude at the end of 2 hours after reactor shutdown.

The particle fall distance and particle density have no significant effects on the fission product release.

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APPENDIX

COMPUTER PROGRAMS FOR THE ESTIMATION OF THE FISSION PRODUCT
FROM A WATER-COOLED REACTOR

The program "RELEASE" is written to calculate the following variables as a function of time:

1. the fraction of UO_2 core vaporized.
2. the fraction of specific fission product vaporized.
3. the airborne inventories of the specific fission product in the pressure vessel, containment, reactor building, and access tunnel.
4. the fraction, the amount and the rate of the specific fission product released to the environment.

Another program "FEED" is written to calculate the effect of fission product vaporization on the decay heat. The correction factor of the decay heat as a function of time obtained by program "FEED" is used as part of the input of program "RELEASE". In this report, the program "RELEASE" is discussed first, then followed by a discussion of program "FEED".

I. THE DESCRIPTION OF THE USAGE OF THE PROGRAM "RELEASE"

The fission product release after a reactor accident depends upon the reactor type and the degree of damage to the reactor containments. Twenty representative cases are built in this code. In all the cases except cases 1, 15, 16, 17, 18, 19, and 20, the standard parametric values are used. The standard parametric values are determined by specifying whether the reactor is a PWR or BWR, to what degree the reactor containments are damaged, and how many containments the

reactor has. Those standard parametric values are listed in Table 8, while the descriptions of the thirteen standard cases are tabulated in Table 9. Cases 1, 15, 16, 17, 18, 19, and 20, nonstandard parametric values, are listed in Table 10. Descriptions of the corresponding nonstandard cases are given in Table 11. In other words, these seven cases are used to investigate the relative effects of the various parameters of the model by comparing the results of those cases with that of case 2.

The notation of some parameters in this code are described briefly in the following and can be changed by the user to get the results of a specific case which is not included in the twenty representative cases.

- a. EMU = total mass of UO_2 core ($1 = 1.48 \times 10^8$ gm in this code).
- b. PWR = the reactor operating power level ($= 3040$ MW in this code).
- c. IC = the number of reactor containments.
- d. H = assumed fraction of decay heat available for the vaporization of UO_2 and fission products.
- e. VP, VC, VB, VT = volumes of pressure vessel, containment, reactor building, and access tunnel.
- f. HP, HC, HB, HT = fall distance of aerosol particles in pressure vessel, containment, reactor building, and access tunnel.
- g. FLTIC, FLTIB, FLTIT = flow rates through iodine absorber in containment, reactor building, and access tunnel.

TABLE 8
STANDARD PARAMETRIC VALUES USED IN THE CODE

Description of the Reactor and Accident Type	Design Basis BWR	Damaged BWR	Design Basis PWR	Damaged PWR
<u>Containment Volume (cm³)</u>				
Pressure Vessel	5x10 ⁸	5x10 ⁸	5x10 ⁸	5x10 ⁸
Containment	5x10 ⁹	5x10 ⁹	6x10 ¹⁰	6x10 ¹⁰
Reactor Building	5x10 ¹⁰	5x10 ¹⁰	--	--
Access Tunnel	5x10 ¹⁰	5x10 ¹⁰	5x10 ¹⁰	5x10 ¹⁰
<u>Filter Adsorber</u>				
Flow Rate (containment volume/sec)	10 ⁻²	10 ⁻²	10 ⁻²	10 ⁻²
Fractional Remove Efficiency	95%	95%	95%	95%
<u>Aerosol Parameters</u>				
Max. Particle Radius (m μ)	10	10	10	10
Min. Particle Radius (m μ)	0.09	0.09	0.09	0.09
Particle Density (g/cm ³)	11	11	11	11
<u>Particle Fall Distance (cm)</u>				
in Pressure Vessel	1x10 ²	1x10 ²	1x10 ²	1x10 ²
in Containment	2x10 ²	2x10 ²	1x10 ³	1x10 ³
in Reactor Building	1x10 ³	1x10 ³	--	--
in Access Tunnel	6x10 ²	6x10 ²	6x10 ²	6x10 ²
Fraction of Decay Heat which Vaporizes UO ₂ Core	10 ⁻³	10 ⁻³	10 ⁻³	10 ⁻³

TABLE 8 (continued)

Description of the Reactor and Accident Type	Design Basis BWR	Damaged BWR	Design Basis PWR	Damaged PWR
<u>Blowdown-Core Heat Up Schedule</u>				
Time for Blowdown (sec)	10	10	10	10
Time for Heat Up (sec)	50	50	50	50
<u>Leak Rate (volume/sec)</u>				
from Pressure Vessel	1x10 ⁻⁴	1x10 ⁻⁴	1x10 ⁻⁴	1x10 ⁻⁴
from Containment	6x10 ⁻⁸	1x10 ⁻⁴	6x10 ⁻⁸	1x10 ⁻⁴
from Reactor Building	1x10 ⁻⁵	1x10 ⁻⁴	--	--
from Access Tunnel	6x10 ⁻⁸	1x10 ⁻⁴	6x10 ⁻⁸	1x10 ⁻⁴

TABLE 9
STANDARD REACTOR ACCIDENT TYPES

Case Number	Reactor Type	Pressure Vessel	Containment	Containment Filter	Building	Access Tunnel
2	BWR	B	B	---	B	---
3	BWR	B	G	---	G	---
4	BWR	B	B	---	B	B
5	PWR	B	B	B	---	---
6	PWR	B	G	G	---	---
7	BWR	B	B	---	B	G
8	BWR	B	G	---	G	B
9	BWR	B	G	---	G	G
10	PWR	B	G	G	---	G
11	BWR	B	B	---	G	---
12	BWR	B	G	---	B	---
13	PWR	B	B	G	---	---
14	PWR	B	G	B	---	---

Where: G = design basis containment

B = damaged containment

--- = does not apply

TABLE 10
 THE NONSTANDARD PARAMETERS USED IN CASES
 1, 15, 16, 17, 18, 19, AND 20

Case Number	Description	Nonstandard Parameter	Parameter Value
1	Higher vaporization rate	Fraction of decay heat which vaporized UO ₂ core	10 ⁻²
15	Lower vaporization rate	Same as in case 1	10 ⁻⁴
16	Larger fall distance	Fall distance (cm) in pressure vessel	10 ³
		In containment	2 x 10 ³
		In reactor building	10 ⁴
17	Larger particle	Minimum particle radius (μ)	0.9
18	Lower particle density	Particle density (g/cm ³)	5
19	Longer blowdown time	Blowdown time (sec)	600
20	Same as case 19	Same as case 19	3600

TABLE 11
NONSTANDARD REACTOR ACCIDENT TYPES

Case Number	Reactor Type	Pressure Vessel	Containment	Containment Filter	Reactor Building	Access Tunnel
1	BWR	B	B	---	B	---
15	BWR	B	B	---	B	---
16	BWR	B	B	---	B	---
17	BWR	B	B	---	B	---
18	BWR	B	B	---	B	---
19	BWR	B	B	---	B	---
20	BWR	B	B	---	B	---

Where: B = damaged containment

--- = does not apply

- h. FLTRC, FLTRB, FLTRT = flow rates through particulate filters in containment, reactor building, and access tunnel.
- i. ELP, ELC, ELB, ELT = leak rates from pressure vessel, containment, reactor building, and access tunnel.

Input of Program "RELEASE"

1. List = IPLOT

Format = I10

IPLOT = 0, no curve plotted.

= 1, plot the curves of fission product inventories in core, pressure vessel, containment, reactor building, access tunnel and the environment as a function of time on a log-log scale.

2. List = ICOL, IROW, NDIM, EPS

Format = (3I10, E10.2)

ICOL = 1.

IROW = number of times at which the correction factor of decay heat is available.

NDIM = number of points used to interpolate the correction factor of decay heat at a given time. (NDIM = 2 means linear interpolation).

EPS = an input constant which is used as the upper bound for the absolute error.

3. List = Z, F

Format = (8D10.4)

Z = values of time (with dimensions equal to IROW)

F = correction factor of decay heat corresponding to the

time in Z (with dimensions equal to IROW).

4. List = IM, XLAMDA, QZERO, G, FP, ATNO

Format = (I10, 5E10.2)

IM = the classification number of the specific fission product we want to calculate.

= 1 for noble gases.

= 2 for volatiles.

= 3 for halogens.

= 4 for refractories.

XLAMDA = decay constant of the specific fission product, sec^{-1}

QZERO = inventory of the specific fission product in fuel at time equal to 0.

G = fraction of halogen fission products which remain gaseous.

FP = ratio of the specific fission product vapor pressure to UO_2 vapor pressure.

ATNO = atomic number of the fission product.

5. List = (TITLE(k), k = 1, 10)

Format = (10A8)

TITLE = consists of 20 cards, on each card print the name of the specific fission product followed by the case number. (For example, I-131 CASE 1). The case number must be in the order: 4, 7, 9, 8, 5, 14, 6, 13, 10, 15, 1, 12, 3, 11, 2, 16, 17, 18, 19, 20. These letters will be printed on the curve as the title of each case.

Output of Program "RELEASE"

1. The following results are printed out as a function of time at 40 points, i.e., with equal time interval on log scale from 10 sec to 10^5 sec.

- a. the fraction of reactor core vaporized.
- b. the fraction of fission products vaporized.
- c. the fraction of fission products released.
- d. the fission product inventories in each vessel.
- e. the fission product release rate.
- f. the sum of the fraction of fission products in each region under the condition without deposition and filtering.

2. If I PLOT equals to 1, the fission product inventories in each region are plotted as a function of time on a log-log scale.

Sample Calculation of Program "RELEASE"

The input data cards are shown in Figure 16. That is:

1st card = I PLOT

2nd card = I COL, I ROW, N DIM, E P S

3rd card to 13th card = Z, F

14th card = I M, X L A M D A, Q Z E R O, G, F P, A T N O

15th card to 24th card = T I T L E

The printed out results together with the plotted curve are shown in Figure 17 and Figure 18, respectively. (The total number of cases given by the output is twenty, but only four of them are shown here in order to save pages.)

```

1
1      42      2      1.E-5
0.0      0.10000 020.12590 020.15850 020.19950 020.25120 020.31620 020.3981 002
0.50120 020.63100 020.75430 020.10000 030.12590 030.15850 030.19950 030.2512 003
0.31620 030.39810 030.50120 030.63100 030.79430 030.10000 040.12590 040.1585 004
0.19950 040.25150 040.31620 040.39810 040.50120 040.63100 040.79430 040.1000 005
0.12590 050.15850 050.19950 050.25150 050.31620 050.39810 050.50120 050.6310 005
0.79430 050.10000 060.0      0.10000 010.99360 000.96840 000.91540 000.8313 000
0.73000 000.64480 000.60000 000.58430 000.57890 000.57810 000.58050 000.5852 000
0.59150 000.59840 000.60540 000.61190 000.61790 000.62280 000.62700 000.6302 000
0.63270 000.63430 000.63530 000.63580 000.63570 000.63560 000.63580 000.6366 000
0.63840 000.64080 000.64330 000.64500 000.64510 000.64410 000.64220 000.6403 000
0.63930 000.63940 000.64050 000.64230 00

```

```

4      8.38E-9      1.20E+7      1.      70.      147.

```

- PM-147 CASE 4
- PM-147 CASE 7
- PM-147 CASE 0
- PM-147 CASE 8
- PM-147 CASE 5
- PM-147 CASE 14
- PM-147 CASE 6
- PM-147 CASE 13
- PM-147 CASE 10
- PM-147 CASE 15
- PM-147 CASE 1
- PM-147 CASE 12
- PM-147 CASE 3
- PM-147 CASE 11
- PM-147 CASE 2
- PM-147 CASE 16
- PM-147 CASE 17
- PM-147 CASE 18
- PM-147 CASE 19
- PM-147 CASE 20

```

/*
//

```

Fig. 16. The Input Data Cards of Program "Release"

CASE 9

PM-147 CASE 9

DECAY CONSTANT, 1/SEC 0.838D-08
 CORE INVENTORY AT ZERO TIME, CURIES 0.120D 08
 RATIO OF FP VAPOR PRESSURE TO UO2 VAPOR PRESSURE 0.700D 02
 FRACTION OF FP WHICH IS GASEOUS 0.100D 01
 CLASS OF FISSION PRODUCT 4

TIME (SEC)	FRACTION OF CORE VAPORIZED	FRACTION OF FP VAPORIZED	FRACTION OF FP RELEASED	INVENTORIES (CURIES)						RELEASE			
				IN CORE	IN PV	IN (2)	IN (3)	IN (4)	(CURIES)	(CI/SEC)			
0.0	0.0	0.0	0.0	0.12D 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.10D 02	0.0	0.0	0.0	0.12D 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.13D 02	0.19D-07	0.13D-05	0.0	0.12D 08	0.13D 02	0.0	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.16D 02	0.97D-07	0.68D-05	0.0	0.12D 08	0.76D 02	0.0	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.20D 02	0.28D-06	0.20D-04	0.0	0.12D 08	0.23D 03	0.0	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.25D 02	0.65D-06	0.46D-04	0.0	0.12D 08	0.53D 03	0.0	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.32D 02	0.13D-05	0.93D-04	0.0	0.12D 08	0.11D 04	0.0	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.40D 02	0.25D-05	0.18D-03	0.0	0.12D 08	0.21D 04	0.61D 00	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.50D 02	0.46D-05	0.32D-03	0.0	0.12D 08	0.38D 04	0.33D 01	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.63D 02	0.78D-05	0.55D-03	0.0	0.12D 08	0.65D 04	0.97D 01	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.79D 02	0.11D-04	0.80D-03	0.0	0.12D 08	0.96D 04	0.23D 02	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.10D 03	0.16D-04	0.11D-02	0.0	0.12D 08	0.13D 05	0.46D 02	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.13D 03	0.21D-04	0.15D-02	0.0	0.12D 08	0.17D 05	0.85D 02	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.16D 03	0.27D-04	0.19D-02	0.0	0.12D 08	0.22D 05	0.15D 03	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.20D 03	0.34D-04	0.24D-02	0.0	0.12D 08	0.28D 05	0.25D 03	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.25D 03	0.43D-04	0.30D-02	0.0	0.12D 08	0.35D 05	0.41D 03	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.32D 03	0.54D-04	0.38D-02	0.0	0.12D 08	0.43D 05	0.67D 03	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.40D 03	0.67D-04	0.46D-02	0.0	0.12D 08	0.53D 05	0.11D 04	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.50D 03	0.82D-04	0.57D-02	0.0	0.12D 08	0.64D 05	0.17D 04	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.63D 03	0.99D-04	0.69D-02	0.0	0.12D 08	0.77D 05	0.26D 04	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.79D 03	0.12D-03	0.84D-02	0.0	0.12D 08	0.92D 05	0.39D 04	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.10D 04	0.14D-03	0.10D-01	0.0	0.12D 08	0.11D 06	0.59D 04	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.13D 04	0.17D-03	0.12D-01	0.0	0.12D 08	0.13D 06	0.90D 04	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.16D 04	0.21D-03	0.14D-01	0.0	0.12D 08	0.15D 06	0.13D 05	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.20D 04	0.25D-03	0.17D-01	0.0	0.12D 08	0.17D 06	0.20D 05	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.25D 04	0.29D-03	0.20D-01	0.0	0.12D 08	0.20D 06	0.29D 05	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.32D 04	0.35D-03	0.24D-01	0.0	0.12D 08	0.22D 06	0.42D 05	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.40D 04	0.41D-03	0.29D-01	0.0	0.12D 08	0.25D 06	0.60D 05	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.50D 04	0.49D-03	0.34D-01	0.0	0.12D 08	0.27D 06	0.85D 05	0.0	0.0	0.0	0.0	0.0	0.10D 01	
0.63D 04	0.58D-03	0.40D-01	0.0	0.12D 08	0.29D 06	0.12D 06	0.54D 01	0.0	0.0	0.0	0.0	0.10D 01	
0.79D 04	0.69D-03	0.47D-01	0.0	0.11D 08	0.31D 06	0.16D 06	0.18D 02	0.0	0.0	0.0	0.0	0.10D 01	
0.10D 05	0.81D-03	0.55D-01	0.0	0.11D 08	0.32D 06	0.21D 06	0.39D 02	0.0	0.0	0.0	0.0	0.10D 01	
0.13D 05	0.96D-03	0.65D-01	0.71D-13	0.11D 08	0.32D 06	0.28D 06	0.75D 02	0.14D 00	0.86D-06	0.70D-08	0.10D 01		
0.16D 05	0.11D-02	0.77D-01	0.24D-10	0.11D 08	0.32D 06	0.35D 06	0.13D 03	0.31D 01	0.29D-03	0.18D-06	0.10D 01		
0.20D 05	0.13D-02	0.90D-01	0.15D-09	0.11D 08	0.31D 06	0.43D 06	0.22D 03	0.96D 01	0.18D-02	0.57D-06	0.10D 01		

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Fig. 17a. The Printed Out Results of Program "Release"

0.250	05	0.160-02	0.110	00	0.550-09	0.110	08	0.300	06	0.510	06	0.340	03	0.220	02	0.660-02	0.130-05	0.100	01	
0.320	05	0.190-02	0.120	00	0.160-08	0.110	08	0.280	06	0.590	06	0.520	03	0.470	02	0.200-01	0.280-05	0.100	01	
0.400	05	0.220-02	0.140	00	0.440-08	0.100	08	0.260	06	0.650	06	0.760	03	0.890	02	0.530-01	0.530-05	0.100	01	
0.500	05	0.260-02	0.170	00	0.110-07	0.100	08	0.240	06	0.690	06	0.110	04	0.160	03	0.130	00	0.960-05	0.100	01
0.630	05	0.300-02	0.190	00	0.250-07	0.970	07	0.220	06	0.710	06	0.140	04	0.270	03	0.300	00	0.160-04	0.100	01
0.790	05	0.360-02	0.220	00	0.530-07	0.930	07	0.200	06	0.700	06	0.180	04	0.430	03	0.640	00	0.260-04	0.100	01
0.100	06	0.420-02	0.260	00	0.110-06	0.890	07	0.180	06	0.660	06	0.220	04	0.650	03	0.130	01	0.390-04	0.100	01

CASE 2

PM-147 CASE 2

DECAY CONSTANT, 1/SEC	0.8380-08
CORE INVENTORY AT ZERO TIME, CURIES	0.1200 08
RATIO OF FP VAPOR PRESSURE TO UO2 VAPOR PRESSURE	0.7000 02
FRACTION OF FP WHICH IS GASEOUS	0.1000 01
CLASS OF FISSION PRODUCT	4

TIME (SEC)	FRACTION OF FP			INVENTORIES (CURIES)					RELEASE	
	VAPORIZED	VAPORIZED	RELEASED	IN CORE	IN PV	IN (2)	IN (3)	IN (4)	(CURIES)	(CI/SEC)
0.0	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0
0.100 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0
0.130 02	0.190-07	0.130-05	0.0	0.120 08	0.130 02	0.0	0.0	0.0	0.0	0.0
0.160 02	0.970-07	0.680-05	0.0	0.120 08	0.760 02	0.0	0.0	0.0	0.0	0.0
0.200 02	0.280-06	0.200-04	0.0	0.120 08	0.230 03	0.0	0.0	0.0	0.0	0.0
0.250 02	0.650-06	0.460-04	0.0	0.120 08	0.530 03	0.0	0.0	0.0	0.0	0.0
0.320 02	0.130-05	0.930-04	0.0	0.120 08	0.110 04	0.0	0.0	0.0	0.0	0.0
0.400 02	0.250-05	0.180-03	0.0	0.120 08	0.210 04	0.0	0.0	0.0	0.0	0.0
0.500 02	0.460-05	0.320-03	0.0	0.120 08	0.380 04	0.0	0.0	0.0	0.0	0.0
0.630 02	0.780-05	0.550-03	0.0	0.120 08	0.650 04	0.390 01	0.0	0.0	0.0	0.0
0.790 02	0.110-04	0.800-03	0.0	0.120 08	0.960 04	0.140 02	0.0	0.0	0.0	0.0
0.100 03	0.160-04	0.110-02	0.0	0.120 08	0.130 05	0.350 02	0.0	0.0	0.0	0.0
0.130 03	0.210-04	0.150-02	0.0	0.120 08	0.170 05	0.700 02	0.0	0.0	0.0	0.0
0.160 03	0.270-04	0.190-02	0.0	0.120 08	0.220 05	0.130 03	0.0	0.0	0.0	0.0
0.200 03	0.340-04	0.240-02	0.0	0.120 08	0.280 05	0.230 03	0.0	0.0	0.0	0.0
0.250 03	0.430-04	0.300-02	0.0	0.120 08	0.350 05	0.390 03	0.0	0.0	0.0	0.0
0.320 03	0.540-04	0.380-02	0.0	0.120 08	0.430 05	0.630 03	0.0	0.0	0.0	0.0
0.400 03	0.670-04	0.460-02	0.130-10	0.120 08	0.530 05	0.100 04	0.0	0.440 00	0.150-03	0.410-04
0.500 03	0.820-04	0.570-02	0.460-08	0.120 08	0.640 05	0.160 04	0.0	0.990 01	0.550-01	0.980-03
0.630 03	0.990-04	0.690-02	0.260-07	0.120 08	0.770 05	0.250 04	0.0	0.320 02	0.320 00	0.320-02
0.790 03	0.120-03	0.840-02	0.980-07	0.120 08	0.920 05	0.380 04	0.0	0.770 02	0.120 01	0.770-02

Fig. 17b.

0.100	04	0.140-03	0.100-01	0.300-06	0.120	08	0.110	06	0.570	04	0.0	0.170	03	0.360	01	0.170-01	0.100	01	
0.130	04	0.170-03	0.120-01	0.840-06	0.120	08	0.130	06	0.850	04	0.0	0.340	03	0.100	02	0.340-01	0.100	01	
0.160	04	0.210-03	0.140-01	0.220-05	0.120	08	0.150	06	0.130	05	0.0	0.660	03	0.260	02	0.660-01	0.100	01	
0.200	04	0.250-03	0.170-01	0.540-05	0.120	08	0.170	06	0.180	05	0.0	0.120	04	0.640	02	0.120	00	0.100	01
0.250	04	0.290-03	0.200-01	0.130-04	0.120	08	0.200	06	0.260	05	0.0	0.230	04	0.150	03	0.230	00	0.100	01
0.320	04	0.350-03	0.240-01	0.300-04	0.120	08	0.220	06	0.370	05	0.0	0.410	04	0.360	03	0.410	00	0.100	01
0.400	04	0.410-03	0.290-01	0.680-04	0.120	08	0.250	06	0.520	05	0.0	0.720	04	0.820	03	0.720	00	0.100	01
0.500	04	0.490-03	0.340-01	0.150-03	0.120	08	0.270	06	0.710	05	0.0	0.120	05	0.180	04	0.120	01	0.100	01
0.630	04	0.580-03	0.400-01	0.330-03	0.120	08	0.290	06	0.940	05	0.0	0.210	05	0.390	04	0.210	01	0.100	01
0.790	04	0.690-03	0.470-01	0.690-03	0.110	08	0.310	06	0.120	06	0.0	0.330	05	0.830	04	0.330	01	0.100	01
0.100	05	0.810-03	0.550-01	0.140-02	0.110	08	0.320	06	0.150	06	0.0	0.510	05	0.170	05	0.510	01	0.100	01
0.130	05	0.960-03	0.650-01	0.280-02	0.110	08	0.320	06	0.180	06	0.0	0.750	05	0.330	05	0.740	01	0.100	01
0.160	05	0.110-02	0.770-01	0.520-02	0.110	08	0.320	06	0.200	06	0.0	0.100	06	0.620	05	0.100	02	0.100	01
0.200	05	0.130-02	0.900-01	0.930-02	0.110	08	0.300	06	0.220	06	0.0	0.130	06	0.110	06	0.130	02	0.100	01
0.250	05	0.160-02	0.110	0.160-01	0.110	08	0.280	06	0.230	06	0.0	0.160	06	0.190	06	0.160	02	0.100	01
0.320	05	0.190-02	0.120	0.250-01	0.110	08	0.260	06	0.220	06	0.0	0.180	06	0.300	06	0.180	02	0.100	01
0.400	05	0.220-02	0.140	0.370-01	0.100	08	0.230	06	0.210	06	0.0	0.180	06	0.450	06	0.180	02	0.100	01
0.500	05	0.260-02	0.170	0.520-01	0.100	08	0.210	06	0.190	06	0.0	0.170	06	0.630	06	0.170	02	0.100	01
0.630	05	0.300-02	0.190	0.700-01	0.970	07	0.190	06	0.160	06	0.0	0.150	06	0.840	06	0.150	02	0.100	01
0.790	05	0.360-02	0.220	0.890-01	0.930	07	0.170	06	0.140	06	0.0	0.130	06	0.110	07	0.130	02	0.100	01
0.100	06	0.420-02	0.260	0.110	0.890	07	0.150	06	0.130	06	0.0	0.110	06	0.130	07	0.110	02	0.100	01

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PM-147 CASE 17

CASE 17

DECAY CONSTANT, 1/SEC 0.8380-08
 CORE INVENTORY AT ZERO TIME, CURIES 0.1200 08
 RATIO OF FP VAPOR PRESSURE TO UO2 VAPOR PRESSURE 0.7000 02
 FRACTION OF FP WHICH IS GASEOUS 0.1000 01
 CLASS OF FISSION PRODUCT 6

TIME (SEC)	FRACTION OF CORE VAPORIZED	FRACTION OF FP VAPORIZED	FRACTION OF FP RELEASED	INVENTORIES (CURIES)					RELEASE		
				IN CORE	IN PV	IN (2)	IN (3)	IN (4)	(CURIES)	(CI/SEC)	
0.0	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01
0.100	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01
0.130	0.190-07	0.130-05	0.0	0.120 08	0.160 02	0.180-01	0.0	0.0	0.0	0.0	0.100 01
0.160	0.970-07	0.680-05	0.0	0.120 08	0.810 02	0.180-01	0.0	0.0	0.0	0.0	0.100 01
0.200	0.280-06	0.200-04	0.0	0.120 08	0.230 03	0.780-01	0.0	0.0	0.0	0.0	0.100 01
0.250	0.650-06	0.460-04	0.0	0.120 08	0.540 03	0.270 00	0.0	0.0	0.0	0.0	0.100 01
0.320	0.130-05	0.930-04	0.0	0.120 08	0.110 04	0.800 00	0.0	0.0	0.0	0.0	0.100 01
0.400	0.250-05	0.180-03	0.0	0.120 08	0.210 04	0.200 01	0.0	0.0	0.0	0.0	0.100 01
0.500	0.460-05	0.320-03	0.0	0.120 08	0.370 04	0.490 01	0.0	0.0	0.0	0.0	0.100 01

Fig. 17c.

0.630 02	0.780-05	0.550-03	0.0	0.120 08	0.620 C4	0.110 02	0.0	0.0	0.0	0.0	0.100 01
0.790 02	0.110-04	0.800-03	0.0	0.120 08	0.890 C4	0.230 02	0.0	0.0	0.0	0.0	0.100 01
0.100 03	0.160-04	0.110-02	0.0	0.120 08	0.120 C5	0.430 02	0.0	0.0	0.0	0.0	0.100 01
0.130 03	0.210-04	0.150-02	0.120-11	0.120 08	0.150 C5	0.760 02	0.0	0.400-01	0.140-04	0.400-05	0.100 01
0.160 03	0.270-04	0.190-02	0.470-10	0.120 08	0.180 05	0.130 03	0.0	0.320 00	0.560-03	0.320-04	0.100 01
0.200 03	0.340-04	0.240-02	0.250-09	0.120 08	0.220 C5	0.200 03	0.0	0.920 00	0.300-02	0.920-04	0.100 01
0.250 03	0.430-04	0.300-02	0.890-09	0.120 08	0.250 C5	0.300 03	0.0	0.210 01	0.110-01	0.210-03	0.100 01
0.320 03	0.540-04	0.380-02	0.260-08	0.120 08	0.280 05	0.430 03	0.0	0.430 01	0.310-01	0.430-03	0.100 01
0.400 03	0.670-04	0.460-02	0.680-08	0.120 08	0.310 05	0.610 03	0.0	0.820 01	0.820-01	0.820-03	0.100 01
0.500 03	0.820-04	0.570-02	0.170-07	0.120 08	0.330 C5	0.820 03	0.0	0.150 02	0.200 00	0.150-02	0.100 01
0.630 03	0.990-04	0.690-02	0.380-07	0.120 08	0.330 C5	0.100 04	0.0	0.250 02	0.460 00	0.250-02	0.100 01
0.790 03	0.120-03	0.840-02	0.830-07	0.120 08	0.330 C5	0.130 04	0.0	0.410 02	0.100 01	0.410-02	0.100 01
0.100 04	0.140-03	0.100-01	0.170-06	0.120 08	0.320 05	0.150 04	0.0	0.630 02	0.210 01	0.630-02	0.100 01
0.130 04	0.170-03	0.120-01	0.340-06	0.120 08	0.300 C5	0.160 04	0.0	0.920 02	0.410 01	0.920-02	0.100 01
0.160 04	0.210-03	0.140-01	0.630-06	0.120 08	0.280 C5	0.170 04	0.0	0.120 03	0.760 01	0.120-01	0.100 01
0.200 04	0.250-03	0.170-01	0.110-05	0.120 08	0.260 C5	0.170 04	0.0	0.160 03	0.130 02	0.160-01	0.100 01
0.250 04	0.290-03	0.200-01	0.190-05	0.120 08	0.240 C5	0.160 04	0.0	0.190 03	0.230 02	0.190-01	0.100 01
0.320 04	0.350-03	0.240-01	0.300-05	0.120 08	0.220 C5	0.150 04	0.0	0.210 03	0.360 02	0.210-01	0.100 01
0.400 04	0.410-03	0.290-01	0.440-05	0.120 08	0.210 C5	0.140 04	0.0	0.220 03	0.530 02	0.220-01	0.100 01
0.500 04	0.490-03	0.340-01	0.630-05	0.120 08	0.190 C5	0.120 04	0.0	0.210 03	0.760 02	0.210-01	0.100 01
0.630 04	0.580-03	0.400-01	0.850-05	0.120 08	0.180 05	0.110 04	0.0	0.200 03	0.100 03	0.200-01	0.100 01
0.790 04	0.690-03	0.470-01	0.110-04	0.110 08	0.160 C5	0.100 04	0.0	0.190 03	0.130 03	0.190-01	0.100 01
0.100 05	0.810-03	0.550-01	0.140-04	0.110 08	0.150 C5	0.960 03	0.0	0.170 03	0.170 03	0.170-01	0.100 01
0.130 05	0.960-03	0.650-01	0.180-04	0.110 08	0.140 05	0.880 03	0.0	0.150 03	0.210 03	0.150-01	0.100 01
0.160 05	0.110-02	0.770-01	0.220-04	0.110 08	0.130 05	0.810 03	0.0	0.140 03	0.260 03	0.140-01	0.100 01
0.200 05	0.130-02	0.900-01	0.260-04	0.110 08	0.120 C5	0.750 03	0.0	0.130 03	0.310 03	0.130-01	0.100 01
0.250 05	0.160-02	0.110 00	0.310-04	0.110 08	0.110 05	0.680 03	0.0	0.110 03	0.380 03	0.110-01	0.100 01
0.320 05	0.190-02	0.120 00	0.370-04	0.110 08	0.100 05	0.620 03	0.0	0.100 03	0.450 03	0.100-01	0.100 01
0.400 05	0.220-02	0.140 00	0.440-04	0.100 08	0.910 04	0.570 03	0.0	0.940 02	0.530 03	0.940-02	0.100 01
0.500 05	0.260-02	0.170 00	0.520-04	0.100 08	0.830 04	0.520 03	0.0	0.850 02	0.620 03	0.850-02	0.100 01
0.630 05	0.300-02	0.190 00	0.600-04	0.970 C7	0.750 04	0.470 03	0.0	0.770 02	0.720 03	0.770-02	0.100 01
0.790 05	0.360-02	0.220 00	0.700-04	0.930 C7	0.670 C4	0.420 03	0.0	0.690 02	0.840 03	0.690-02	0.100 01
0.100 06	0.420-02	0.260 00	0.810-04	0.890 C7	0.600 04	0.380 03	0.0	0.620 02	0.980 03	0.620-02	0.100 01

Fig. 17d.

PM-147 CASE 20

CASE 20

DECAY CONSTANT, 1/SEC 0.8380-08
 CORE INVENTORY AT ZERO TIME, CURIES 0.1200 08
 RATIO OF FP VAPOR PRESSURE TO UO2 VAPOR PRESSURE 0.7000 02,
 FRACTION OF FP WHICH IS GASEOUS 0.1000 01
 CLASS OF FISSION PRODUCT 4

TIME (SEC)	FRACTION OF CORE VAPORIZED	FRACTION OF FP VAPORIZED	FRACTION OF FP RELEASED	INVENTORIES (CURIES)					RELEASE				
				IN CCR	IN PV	IN (2)	IN (3)	IN (4)	(CURIES)	(CI/SEC)			
0.0	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.100 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.130 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.160 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.200 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.250 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.320 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.400 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.500 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.630 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.790 02	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.100 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.130 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.160 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.200 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.250 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.320 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.400 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.500 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.630 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.790 03	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.100 04	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.130 04	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.160 04	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.200 04	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.250 04	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.320 04	0.0	0.0	0.0	0.120 08	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.100 01	
0.400 04	0.280-04	0.190-02	0.200-08	0.120 08	0.230 05	0.400 03	0.0	0.340 01	0.240-01	0.330-03	0.100 01		
0.500 04	0.100-03	0.720-02	0.720-06	0.120 08	0.790 05	0.540 04	0.0	0.240 03	0.860 01	0.240-01	0.100 01		
0.630 04	0.190-03	0.130-01	0.920-05	0.120 08	0.130 06	0.170 05	0.0	0.150 04	0.110 03	0.150 00	0.100 01		
0.790 04	0.300-03	0.210-01	0.530-04	0.120 08	0.180 06	0.380 05	0.0	0.540 04	0.640 03	0.540 00	0.100 01		
0.100 05	0.430-03	0.290-01	0.210-03	0.120 08	0.230 06	0.660 05	0.0	0.140 05	0.250 04	0.140 01	0.100 01		

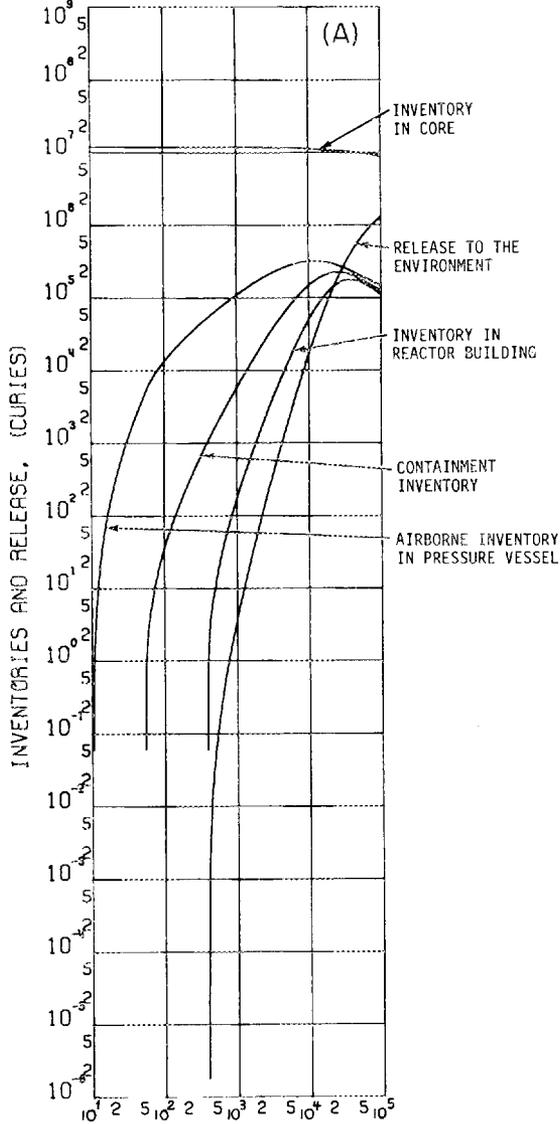
70

Fig. 17e.

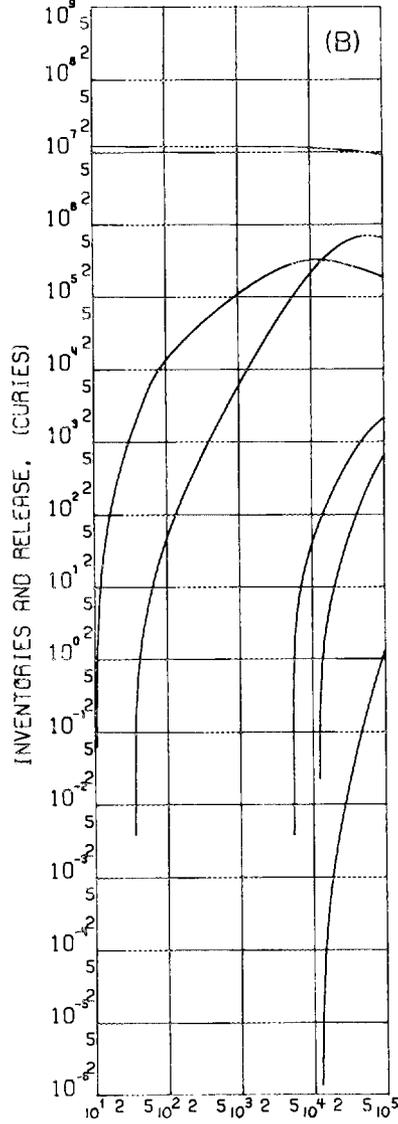
0.130 05	0.570-03	0.390-01	0.670-03	0.120 08	0.260 06	0.100 06	0.0	0.290 05	0.800 04	0.290 01	0.100 01
0.160 05	0.750-03	0.510-01	0.180-02	0.110 08	0.280 06	0.140 06	0.0	0.530 05	0.210 05	0.530 01	0.100 01
0.200 05	0.960-03	0.650-01	0.410-02	0.110 08	0.280 06	0.170 06	0.0	0.840 05	0.500 05	0.840 01	0.100 01
0.250 05	0.120-02	0.800-01	0.850-02	0.110 08	0.270 06	0.200 06	0.0	0.120 06	0.100 06	0.120 02	0.100 01
0.320 05	0.150-02	0.980-01	0.160-01	0.110 08	0.260 06	0.210 06	0.0	0.150 06	0.190 06	0.150 02	0.100 01
0.400 05	0.180-02	0.120 00	0.270-01	0.110 08	0.240 06	0.200 06	0.0	0.160 06	0.320 06	0.160 02	0.100 01
0.500 05	0.220-02	0.140 00	0.410-01	0.100 08	0.210 06	0.190 06	0.0	0.170 06	0.490 06	0.170 02	0.100 01
0.630 05	0.260-02	0.170 00	0.530-01	0.100 08	0.190 06	0.170 06	0.0	0.150 06	0.700 06	0.150 02	0.100 01
0.790 05	0.320-02	0.200 00	0.780-01	0.960 07	0.170 06	0.150 06	0.0	0.140 06	0.930 06	0.140 02	0.100 01
0.100 06	0.380-02	0.240 00	0.990-01	0.920 07	0.150 06	0.130 06	0.0	0.120 06	0.120 07	0.120 02	0.100 01

Fig. 17f.

PM-147 CASE 2



PM-147 CASE 9



TIME AFTER CESSATION OF FISSIONING, (SEC)

TIME AFTER CESSATION OF FISSIONING, (SEC)

Fig. 18. The Plotted Curves of Program "Release"

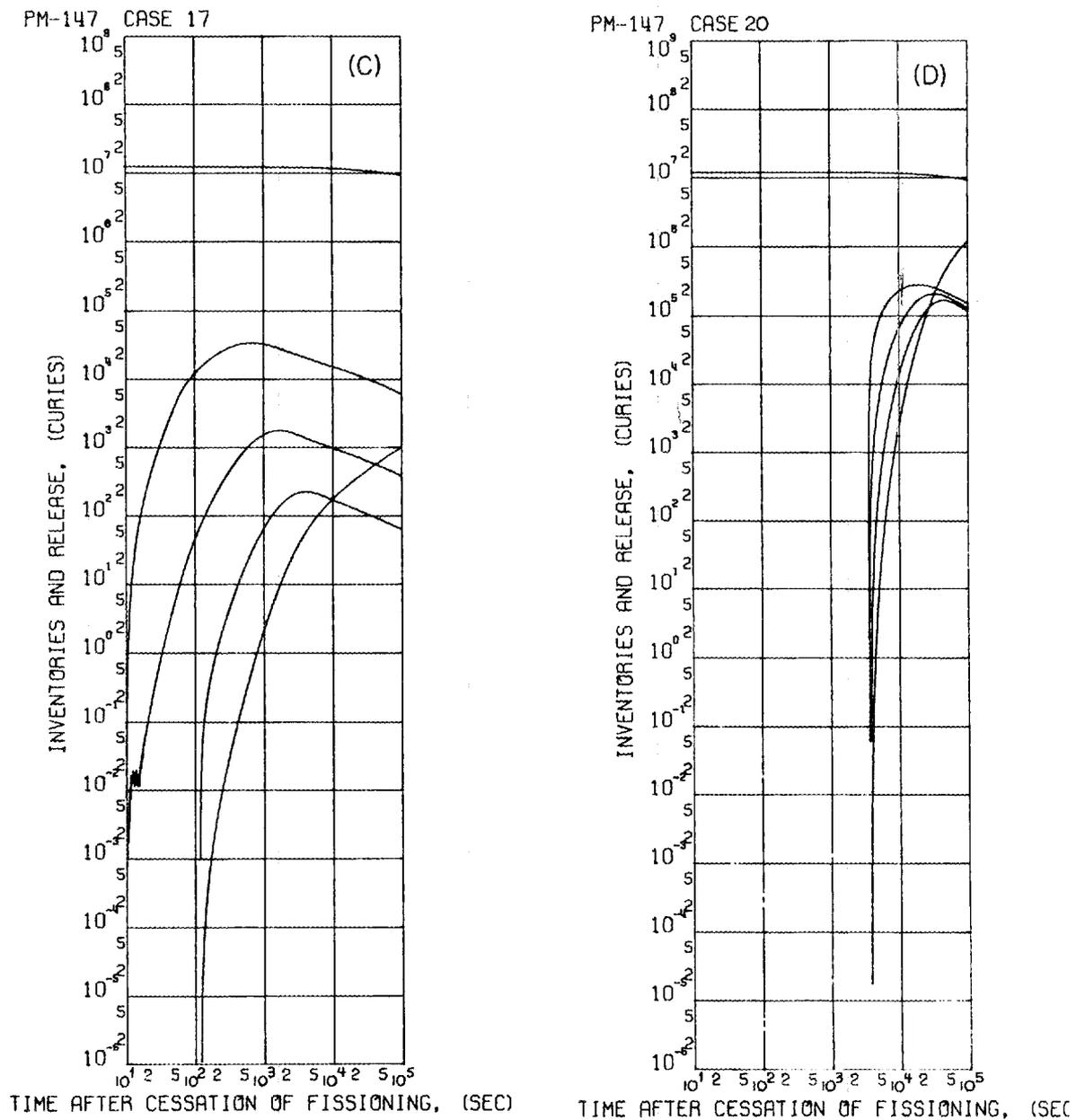


Fig. 19. The Input Cards of Program "Feed"

II. PROGRAM "FEED"

A. Input

1. List = ICOL, IROW, NDIM, EPS, ISET

Format = (3I10, E10.2, I10)

ICOL = 1

IROW = number of times at which the F value (will be explained later) is available.

NDIM = number of points used to interpolate the F value at a given time. (= 2 means linear interpolation).

EPS = an input constant which is used as the upper bound for the absolute error.

ISET = this value determines the total number of runs. In this code ISET must be less than 20. By our calculation, if $H = 10^{-2}$, ISET = 7 is enough to get the convergent result; for $H = 10^{-4}$, ISET = 4 is enough.

2. List = IM, XLAMDA, QZERO, FP

Format = (I10, 3E10.2)

IM = The group number of the specific fission product

= 1 for As, Se, Br, Kr, Rb, Cd, I, Xe, Cs

= 2 for Ge, Sr, Pd, Sn

= 3 for Ag, In

= 4 for Sb, Ba

= 5 for La, Nd, Pm

= 6 for Ge, Pr, Te

= 7 for Rh, Sm

= 8 for Y, Nb

= 9 for Zr, Ru

= 10 for Mo, Tc

XLAMDA = decay constant of the fission product, sec^{-1}

QZERO = can be set to arbitrary value.

FP = ratio of fission product vapor pressure to UO_2 vapor pressure. In our classification, the FP values for the ten groups are as follows:

group 1	1.0×10^6
group 2	8.0×10^2
group 3	1.0×10^4
group 4	1.5×10^5
group 5	70.0
group 6	8.0
group 7	1.0
group 8	0.25
group 9	3.0×10^{-2}
group 10	7.0×10^{-3}

3. List = Z

Format = (8E10,4)

Z = values of times at which the F value is available
(with dimensions equal to IROW).

4. List = F

Format = (8Z10.4)

F = the fraction of total decay heat attributable to all the fission products in one classification at the corresponding time in Z.

The IM, XLAMDA, QZERO, FP, Z, F input data cards of fission product group one is followed by that of fission product group two and then followed by that of fission product group three and so on.

The most important parameter which is subjected to change in this code is the assumed fraction of decay heat available for the vaporization of UO_2 and fission products (denoted by H in the code). The other two parameters which are subjected to change are the total mass of UO_2 (denoted by EMU) and the reactor operation level (denoted by PWR), but both are not as important as the H value.

B. Output

For each run the fraction of fission products vaporized as a function of time is printed out for each of the ten fission product groups. Then the correction factor is printed out also as a function of time. All of these are printed out at equal time intervals (log scale) between 10 sec and 10^6 sec.

The notations on the output are explained as follows:

INDI = This number indicates the number of runs. For example, on the second run INDI = 2 is printed at the top of each page.

IM = The fission product group number.

MULTIPLIER = the correction factor.

C. Sample Calculation

The input data cards are shown in Figure 19. That is:

1st card = ICOL, IROW, NDIM, EPS, ISET

2nd card = IM, XLAMDA, QZERO, FP

3rd card to 7th card = Z of fission product group one

8th card to 12th card = F of fission product group one

The same pattern of input cards repeated for each of the ten fission product groups and the output is shown in Table 12. In order to save space only part of the 4th run is shown here.

A list of the computer programs "RELEASE" and "FEED" follows.

ICOL = 1
IROW = 34
NDIM = 2
EPS = 0.10E-03
ISET = 10

IM = 1
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.10E 07

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.3806E 000.3756E 000.3698E 000.3633E 000.3560E 000.3481E 000.3397E 000.3311E 00
.3226E 000.3145E 000.3073E 000.3010E 000.2957E 000.2916E 000.2884E 000.2862E 00
.2848E 000.2840E 000.2837E 000.2833E 000.2823E 000.2800E 000.2760E 000.2706E 00
.2644E 000.2585E 000.2541E 000.2511E 000.2489E 000.2462E 000.2418E 000.2351E 00
.2258E 000.2142E 00

IM = 2
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.80E 03

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.5856E-010.5897E-010.5943E-010.5997E-010.6059E-010.6128E-010.6206E-010.6293E-01
.6390E-010.6498E-010.6619E-010.6755E-010.6909E-010.7082E-010.7275E-010.7485E-01
.7710E-010.7944E-010.8175E-010.8392E-010.8580E-010.8724E-010.8811E-010.8821E-01
.8736E-010.8534E-010.8209E-010.7781E-010.7288E-010.6777E-010.6281E-010.5819E-01
.5397E-010.5031E-01

IM = 3
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.10E 05

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
.0	0.0						

IM = 4
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.15E 06

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.6471E-010.6524E-010.6584E-010.6651E-010.6724E-010.6803E-010.6883E-010.6959E-01
.7026E-010.7076E-010.7101E-010.7093E-010.7044E-010.6949E-010.6802E-010.6601E-01
.6346E-010.6037E-010.5683E-010.5298E-010.4902E-010.4516E-010.4160E-010.3845E-01
.3580E-010.3375E-010.3240E-010.3183E-010.3209E-010.3308E-010.3463E-010.3655E-01
.3868E-010.4090E-01

IM = 5
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.70E 02

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.1454E 000.1467E 000.1483E 000.1502E 000.1523E 000.1546E 000.1571E 000.1598E 00
.1626E 000.1654E 000.1681E 000.1705E 000.1727E 000.1745E 000.1760E 000.1769E 00
.1774E 000.1773E 000.1765E 000.1751E 000.1730E 000.1701E 000.1668E 000.1631E 00
.1596E 000.1566E 000.1547E 000.1546E 000.1567E 000.1609E 000.1674E 000.1757E 00
.1857E 000.1969E 00

IM = 6
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.80E 01

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.1404E 000.1414E 000.1425E 000.1437E 000.1449E 000.1462E 000.1476E 000.1488E 00
.1499E 000.1508E 000.1515E 000.1520E 000.1525E 000.1528E 000.1531E 000.1531E 00
.1527E 000.1516E 000.1496E 000.1467E 000.1432E 000.1398E 000.1370E 000.1355E 00
.1355E 000.1369E 000.1394E 000.1428E 000.1468E 000.1512E 000.1561E 000.1612E 00
.1664E 000.1716E 00

IM = 7
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.10E 01

I

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.2613E-020.2638E-020.2668E-020.2702E-020.2741E-020.2786E-020.2835E-020.2889E-02
.2946E-020.3007E-020.3069E-020.3134E-020.3202E-020.3274E-020.3353E-020.3440E 00
.3540E-020.3656E-020.3790E-020.3945E-020.4128E-020.4340E-020.4586E-020.4863E-02

.5166E-020.5482E-020.5800E-020.6116E-020.6433E-020.6762E-020.7109E-020.7468E-02

.7829E-020.8178E-02

IM = 8
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.25E 00

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03

.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04

.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04

.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05

.7554E 050.9509E 05

F

.9174E-010.9273E-010.9389E-010.9526E-010.9684E-010.9865E-010.1007E 000.1030E 00

.1056E 000.1083E 000.1114E 000.1147E 000.1183E 000.1223E 000.1268E 000.1319E 00

.1377E 000.1442E 000.1514E 000.1592E 000.1675E 000.1760E 000.1848E 000.1932E 00

.2007E 000.2063E 000.2091E 000.2086E 000.2049E 000.1986E 000.1910E 000.1829E 00

.1755E 000.1693E 00

.IM = 9
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.30E-01

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.3811E-010.3852E-010.3900E-010.3956E-010.4021E-010.4096E-010.4180E-010.4274E-01
.4378E-010.4492E-010.4615E-010.4748E-010.4894E-010.5056E-010.5238E-010.5442E-01
.5674E-010.5934E-010.6222E-010.6536E-010.6875E-010.7238E-010.7624E-010.8029E-01
.8440E-010.8840E-010.9206E-010.9531E-010.9820E-010.1009E 000.1034E 000.1059E 00
.1081E 000.1101E 00

IM = 10
XLAMDA = 0.10E-06
QZERO = 0.10E 01
FP = 0.70E-02

Z

.4766E 020.6000E 020.7554E 020.9509E 020.1197E 030.1507E 030.1897E 030.2389E 03
.3007E 030.3786E 030.4766E 030.6000E 030.7554E 030.9509E 030.1197E 040.1507E 04
.1897E 040.2389E 040.3007E 040.3786E 040.4766E 040.6000E 040.7554E 040.9509E 04
.1197E 050.1507E 050.1897E 050.2389E 050.3007E 050.3786E 050.4766E 050.6000E 05
.7554E 050.9509E 05

F

.7788E-010.7822E-010.7856E-010.7890E-010.7918E-010.7937E-010.7938E-010.7912E-01
.7846E-010.7727E-010.7539E-010.7271E-010.6913E-010.6461E-010.5921E-010.5309E-01
.4657E-010.4009E-010.3423E-010.2953E-010.2640E-010.2494E-010.2491E-010.2582E-01
.2721E-010.2876E-010.3030E-010.3177E-010.3317E-010.3453E-010.3584E-010.3702E-01
.3793E-010.3843E-01
IHCO02I STOP 00000

TABLE 12
THE PRINTED OUTPUT OF PROGRAM "FEED"

INDI = IM =	10 ⁴ Fraction of FP Vaporized	INDI =	4 Multiplier
Time (Sec)		Time	
0.0	0.0	0.0	0.0
0.1000D 02	0.0	0.1000D 02	0.1000D 01
0.1259D 02	0.1118D-09	0.1259D 02	0.9936D 00
0.1585D 02	0.5706D-09	0.1585D 02	0.9684D 00
0.1995D 02	0.1652D-08	0.1995D 02	0.9154D 00
0.2512D 02	0.3812D-08	0.2512D 02	0.8313D 00
0.3162D 02	0.7798D-08	0.3162D 02	0.7301D 00
0.3981D 02	0.1482D-07	0.3981D 02	0.6448D 00
0.5012D 02	0.2684D-07	0.5012D 02	0.6000D 00
0.6310D 02	0.4680D-07	0.6310D 02	0.5843D 00
0.7943D 02	0.7236D-07	0.7943D 02	0.5789D 00
0.1000D 03	0.1024D-06	0.1000D 03	0.5781D 00
0.1259D 03	0.1383D-06	0.1259D 03	0.5805D 00
0.1585D 03	0.1813D-06	0.1585D 03	0.5852D 00
0.1995D 03	0.2330D-06	0.1995D 03	0.5915D 00
0.2512D 03	0.2951D-06	0.2512D 03	0.5984D 00
0.3162D 03	0.3694D-06	0.3162D 03	0.6054D 00
0.3981D 03	0.4578D-06	0.3981D 03	0.6119D 00
0.5012D 03	0.5627D-06	0.5012D 03	0.6179D 00
0.6310D 03	0.6866D-06	0.6310D 03	0.6228D 00
0.7943D 03	0.8328D-06	0.7943D 03	0.6270D 00
0.1000D 04	0.1005D-05	0.1000D 04	0.6302D 00
0.1259D 04	0.1207D-05	0.1259D 04	0.6327D 00
0.1585D 04	0.1444D-05	0.1585D 04	0.6343D 00
0.1995D 04	0.1723D-05	0.1995D 04	0.6353D 00
0.2512D 04	0.2049D-05	0.2512D 04	0.6358D 00
0.3162D 04	0.2432D-05	0.3162D 04	0.6357D 00
0.3981D 04	0.2883D-05	0.3981D 04	0.6356D 00
0.5012D 04	0.3415D-05	0.5012D 04	0.6358D 00
0.6310D 04	0.4045D-05	0.6310D 04	0.6366D 00
0.7943D 04	0.4795D-05	0.7943D 04	0.6384D 00
0.1000D 05	0.5687D-05	0.1000D 05	0.6408D 00
0.1259D 05	0.6741D-05	0.1259D 05	0.6433D 00
0.1585D 05	0.7977D-05	0.1585D 05	0.6450D 00
0.1995D 05	0.9414D-05	0.1995D 05	0.6451D 00
0.2512D 05	0.1108D-04	0.2512D 05	0.6441D 00
0.3162D 05	0.1303D-04	0.3162D 05	0.6422D 00
0.3981D 05	0.1532D-04	0.3981D 05	0.6403D 00
0.5012D 05	0.1803D-04	0.5012D 05	0.6394D 00
0.6310D 05	0.2125D-04	0.6310D 05	0.6394D 00
0.7943D 05	0.2508D-04	0.7943D 05	0.6405D 00
0.1000D 06	0.2962D-04	0.1000D 06	0.6423D 00

(a)

(b)

```

**FTN,L,E,M,G.
PROGRAM RELEASE
IMPLICIT REAL * 8(A-H,O-Z)
DIMENSION TIME(4550),
1,P(4550),C(4550),B(4550),R(4550),FB(4550),CSAV(4550),BSAV(4550),FB
2SAV(4550),RSAV(4550),W(4550),FU(4550),TITLE(10),RR(4550),RRSAV(455
30),T(4550),TSAV(4550),AY(6)
4,Z(42),F(42),WORK(42),ARG(5),VAL(5)
C CONSTANTS
C TIME AT FIRST CALCULATED VALUES, SEC
TSTART=10.
C NUMBER OF VALUES CALCULATED PER DECADE OF TIME
C FROM T2 TO 1.E2 SEC
XN2=1000.
C FROM 1.E2 TO 1.E3 SEC
XN3=1500.
C FROM 1.E3 TO 1.E4 SEC
XN4=1500.
C FROM 1.E4 TO 1.E5 SEC
XN5=500.
C FROM 1.E5 TO 1.E6 SEC
XN6=20.
C TIME AT LAST CALCULATED CONCENTRATION, SEC
TSTOP=.999E5
C ACCELERATION DUE TO GRAVITY, CM/SEC**2
GEE = 980.
C MEAN FREE PATH OF MOLECULES OF THE MEDIUM, CM
ELL = 6.69E-6
C VISCOSITY OF THE MEDIUM, G/CM-SEC
ETA = 1.82E-4
C CUNNINGHAM CORRECTION CONSTANTS
AA = 0.864

```

```

C DELTA TIME PARAMETER
  U=2.
C TOTAL MASS OF UO2 CORE, G
  EMU = 1.48 E8
C REACTOR POWER, WATTS
  PWR=3.44E9
C FRACTIONAL REMOVAL EFFICIENCY OF IODINE ADSORBER ON CONTAINMENT
  EFFI=0.95
C FRACTIONAL REMOVAL EFFICIENCY OF PARTICLE FILTER ON CONTAINMENT
  EFF=.95
C IPLOT=1 USE PLOT SUBROUTINE
  READ 81, IPLOT
  81 FORMAT (I10)
C ICOL,IROW,NDIM,EPS,Z,F ARE DESCRIBED IN SUBROUTINE DATSG WHICH IS USED
C TO INTERPOLATE THE CORRECTION FACTOR OF THE DECAY HEAT
  READ 7,ICOL,IROW,NDIM,EPS
  7 FORMAT(3I10,E10.2)
  READ 8,Z,F
  8 FORMAT(8D10.4)
C READ CLASS OF FISSION PRODUCT
C READ DECAY CONSTANT, 1/SEC
C READ INVENTORY IN FUEL AT TIME ZERO, CURIES
C READ FRACTION OF CLASS 3 FP WHICH REMAINS GASEOUS
C READ RATIO OF FISSION PRODUCT VAPOR PRESSURE TO UO2 VAPOR PRESSURE
C READ THE ATOMIC NUMBER OF THE FISSION PRODUCT
C LOOP FOR DIFFERENT FISSION PRODUCTS
  200 READ(50,58,END=17) IM,XLAMDA,QZERO,G,FP,ATNO
  58 FORMAT (I10,5E10.2)
C LOOP FOR DIFFERENT CASES
  DO 600 II=1,20
    IF(II.EQ. 1) GO TO 31
    IF(II.EQ. 2) GO TO 32
    IF(II.EQ. 3) GO TO 33
    IF(II.EQ. 4) GO TO 34
    IF(II.EQ. 5) GO TO 35

```

```

IF(II.EQ. 6) GO TO 36
IF(II.EQ. 7) GO TO 37
IF(II.EQ. 8) GO TO 38
IF(II.EQ. 9) GO TO 39
IF(II.EQ.10) GO TO 40
IF(II.EQ.11) GO TO 41
IF(II.EQ.12) GO TO 42
IF(II.EQ.13) GO TO 43
IF(II.EQ.14) GO TO 44
IF(II.EQ.15) GO TO 45
IF(II.EQ.16) GO TO 46
IF(II.EQ.17) GO TO 47
IF(II.EQ.18) GO TO 48
IF(II.EQ.19) GO TO 49
IF(II.EQ.20) GO TO 50
GO TO 600
31 CONTINUE
C CASE 4
C READ TITLE
  READ 16,(TITLE(K),K=1,10)
  III= 4
C TIME AFTER SHUTDOWN AT WHICH VAPORIZATION OF CORE BEGINS, SEC
  T1=60.
C TIME AFTER SHUTDOWN AT WHICH HEAT GENERATION FUNCTION CHANGES, SEC
  T2=10.
C NUMBER OF CONTAINMENTS, INCLUDING PRESSURE VESSEL
  IC=4
C FRACTION OF DECAY HEAT WHICH VAPORIZES UO2
  H=1.E-3
C VOLUME OF PRESSURE VESSEL, CM**3
  VP=5.E8
C VOLUME OF SECOND CONTAINMENT, CM**3
  VC=5.E9
C VOLUME OF THIRD CONTAINMENT, CM**3
  VB=5.E10

```

```
C VOLUME OF LAST CONTAINMENT, CM**3
  VT=5.E10
C FALL DISTANCE FOR PARTICLES IN PRESSURE VESSEL, CM
  HP=1.E2
C FALL DISTANCE FOR PARTICLES IN SECOND CONTAINMENT, CM
  HC=2.E2
C FALL DISTANCE FOR PARTICLES IN THIRD CONTAINMENT, CM
  HB=1.E3
C FALL DISTANCE FOR PARTICLES IN LAST CONTAINMENT, CM
  HT=6.E2
C FLOW RATES THROUGH IODINE ADSORBERS, CM**3/SEC
  FLTIC=0.
  FLTIB=0.
  FLTIT=0.
C FLOW RATES THROUGH PARTICULATE FILTERS, CM**3/SEC
  FLTRC=0.
  FLTRB=0.
  FLTRT=0.
C LEAK RATE FROM PRESSURE VESSEL, CM**3/SEC
  ELP=1.E-4*VP
C LEAK RATE FROM SECOND CONTAINMENT, CM**3/SEC
  ELC=1.E-4*VC
C LEAK RATE FROM THIRD CONTAINMENT, CM**3/SEC
  ELB=1.E-4 *VB
C LEAK RATE FROM LAST CONTAINMENT, CM**3/SEC
  ELT=1.E-4 *VT
C PARTICLE SIZE DISTRIBUTION PARAMETERS
  RHO=11.
  RMIN=9.E-6
  RMAX=1.E-3
  X=4.
  GO TO 75
  32 CONTINUE
C CASE 7
C READ TITLE
```

```

        READ 16,(TITLE(K),K=1,10)
        III= 7
        ELT=6.E-8*VT
        GO TO 75
    33 CONTINUE
C CASE 9
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III= 9
    ELB=1.E-5 *VB
    ELC=6.E-8*VC
    GO TO 75
    34 CONTINUE
C CASE 8
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III= 8
    ELT=1.E-4 *VT
    GO TO 75
    35 CONTINUE
C CASE 5
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III= 5
    ELC=1.E-4*VC
    IC=2
    VC=6.E10
    HC=1.E3
    GO TO 75
    36 CONTINUE
C CASE 14
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III= 14

```

```
      ELC=6.E-8 *VC
      GO TO 75
37 CONTINUE
C CASE 6
C READ TITLE
      READ 16,(TITLE(K),K=1,10)
      III= 6
      FLTIC=1.E-2*VC
      FLTRC=1.E-2*VC
      GO TO 75
38 CONTINUE
C CASE 13
C READ TITLE
      READ 16,(TITLE(K),K=1,10)
      III= 13
      ELC=1.E-4*VC
      GO TO 75
39 CONTINUE
C CASE X
C READ TITLE
      READ 16,(TITLE(K),K=1,10)
16 FORMAT(10A8)
      III=10
      T2=10.
      T1 = 60.
      IC=3
      H=1.E-3
      VP=5.E8
      VC=6.E10
      VB=5.E10
      VT=1.E10
      HP=1.E2
      HC=1.E3
```

```
HB=6.E2
HT=6.E2
FLTIC=1.E-2*VC
FLTIB=0.
FLTIT=0.
FLTRC=1.E-2*VC
FLTRB=0.
FLTRY=0.
ELP=1.E-4*VP
ELC=6.E-8*VC
ELB=6.E-8*VB
ELT=1.E-4*VT
RHO = 11.
176 RMAX=1.E-3
RMIN=9.E-6
X = 4.
GO TO 75
40 CONTINUE
C CASE XV
C READ TITLE
READ 16,(TITLE(K),K=1,10)
III=15
IC=3
H=1.E-4
VP=5.E8
VC=5.E9
VB=5.E10
HP=1.E2
HC=2.E2
HB=1.E3
FLTIC=0.
FLTIB=0.
FLTRC=0.
FLTRB=0.
ELP=1.E-4*VP
ELC=1.E-4*VC
```

```

        ELB=1.E-4*VB
        GO TO 75
    41 CONTINUE
C CASE I
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III= 1
    H=1.E-2
    GO TO 75
    42 CONTINUE
C CASE XII
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III=12
    H=1.E-3
    ELC=6.E-8*VC
    GO TO 75
    43 CONTINUE
C CASE III
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III= 3
    ELB=1.E-5*VB
    GO TO 75
    44 CONTINUE
C CASE XI
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III=11
    ELC=1.E-4*VC
    GO TO 75
    45 CONTINUE
C CASE II
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III= 2

```

```

        ELB=1.E-4*VB
        H=1.E-3
        GO TO 75
    46 CONTINUE
C CASE XVI
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III=16
    H=1.E-3
    HP=1.E3
    HC=2.E3
    HB=1.E4
    GO TO 75
    47 CONTINUE
C CASE XVII
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III=17
    HP=1.E2
    HC=2.E2
    HB=1.E3
    RMIN=9.E-5
    GO TO 75
    48 CONTINUE
C CASE XVIII
C READ TITLE
    READ 16,(TITLE(K),K=1,10)
    III=18
    RMIN=9.E-6
    RHO=5.
    GO TO 75
    49 CONTINUE
C READ TITLE
C CASE XIX
    READ 16,(TITLE(K),K=1,10)
    III=19

```

```

      RHO=11.
      T2=600.
      T1=650.
      GO TO 75
50 CONTINUE
C CASE XX
C READ TITLE
  READ 16,(TITLE(K),K=1,10)
  III=20
  T2=3600.
  T1=3650.
75 CONTINUE
C SET INITIAL VALUES
  DO 110 I=1,4550
C TIME AFTER SHUTDOWN, SEC
  TIME(I) = 0.
C FRACTION OF FISSION PRODUCT VAPORIZED
  FI(I)=0.
C FRACTION OF CORE VAPORIZED
  FU(I)=0.
C INVENTORY OF ONE FISSION PRODUCT IN THE FUEL, CURIES
  Q(I)=0.
C TOTAL INVENTORY OF ONE FISSION PRODUCT AIRBORNE IN THE PRESSURE VESSEL
C ,CURIES
  P(I) = 0.
C TOTAL INVENTORY OF ONE FISSION PRODUCT AIRBORNE IN THE CONTAINMENT,
C CURIES
  C(I) = 0.
C TOTAL INVENTORY OF ONE FISSION PRODUCT AIRBORNE IN THE REACTOR
C BUILDING,CURIES
  B(I) = 0.
C TOTAL INVENTORY OF ONE FISSION PRODUCT AIRBORNE IN THE ACCESS TUNNEL,
C CURIES
  T(I)=0.
C TOTAL INVENTORY OF ONE FISSION PRODUCT AIRBORNE IN THE ENVIRONMENT,
C CURIES
  R(I) = 0.

```

```

C RELEASE RATE OF FISSION PRODUCT INTO THE ENVIRONMET, CURIES/SEC
  RR(I)=0.
C FRACTION OF FISSION PRODUCT RELEASED
  FB(I)=0.
C IN CALCULATING HALOGEN FISSION PRODUCTS  CSAV,BSAV,TSAV,RSAV,RRSAV,
C FBSAV ARE USED TO STORE THE RESULT OF THE FIRST LOOP
  CSAV(I) = 0.
  BSAV(I) = 0.
  TSAV(I)=0.
  RSAV(I) = 0.
  RRSAB(I)=0.
  FBSAV(I)=0.
C W VALUE IS USED TO CHECK THE CALCULATION , IT SHOULD BE EQUAL TO ONE.
C A LOOP TO CALCULATE W(I)
110 W(I)=1.
C CALCULATION OF TIME INDEPENDENT PARAMETERS
  A = 3./((1./(RMIN**3))-(1./(RMAX**3)))
  S = 1./DLOG(RMAX/RMIN)
  E = 1./RMIN-1./RMAX
C RATE OF VAPORIZATION OF UC2 CORE AT TIME T1, G/SEC
  ELUT1=6.511E-5*PWR*DEXP(-.291*DLOG(T1))*H*0.7
201 ELUT1=ELUT1*EXPF(-25.*ELUT1/EMU)
C FRACTION OF CORE VAPORIZED AT TIME T1
  FUT1=ELUT1*(T1-T2)/(2.*EMU)
C COAGULATION RATE CONSTANT
  D = 3.E-10 *RHO*A/S*4.*3.1416/3.*6.023E23/ATNO*XLAMDA/3.7E10
  CPRIME = 1.+(AA*ELL/(E*(X-2.)))*((1./RMIN**2)-(1./RMAX**2))
  VFALL = (2.*RHO*GEE*A*E*CPRIME)/(9.*ETA)
  DNP=VFALL/HP
  DNC=VFALL/HC
  DNB=VFALL/HB
  DNT=VFALL/HT
C CALCULATION OF CORRECTION TERMS, W(I)
  2 IMSAV=IM
  IM=1
  IMM=1
  GO TO 151

```

```

C T LOOP TO CALCULATE FISSION PRODUCT INVENTORIES AND RELEASE RATES
152 IMM=2
    IM=IMSAV
151 IM3=0
    Q(1)=QZERO
    IF((XLAMDA*T2).GT.70.) EX2=0.
    IF((XLAMDA*T2).GT.70.) GO TO 115
    IF((XLAMDA*T2).LT.1.E-6) EX2=1.-XLAMDA*T2
    IF((XLAMDA*T2).LT.1.E-6) GO TO 115
    EX2=DEXP(-XLAMDA*T2)
115 QT2=QZERO*EX2
    X1=-FP*DLOG(1.-FUT1)+XLAMDA*(T1-T2)
    IF(X1.GT.70.) EX1=0.
    IF(X1.GT.70.) GO TO 116
    IF(X1.LT.1.E-6) EX1=1.-X1
    IF(X1.LT.1.E-6) GO TO 116
    EX1=DEXP(-X1)
116 QT1=QT2*EX1
3000 I = 1
    3 I = I + 1
    XI=I-2
    IF(TIME(I-1).LT..999E2) TIME(I)=TSTART*(10.**((XI/XN2)))
    IF(TIME(I-1).GE..999E2) TIME(I)=TIME(I-1)*(10.**((1./XN3)))
    IF(TIME(I-1).GE..999E3) TIME(I)=TIME(I-1)*(10.**((1./XN4)))
    IF(TIME(I-1).GE..999E4) TIME(I)=TIME(I-1)*(10.**((1./XN5)))
    IF(TIME(I-1).GE..999E5) TIME(I)=TIME(I-1)*(10.**((1./XN6)))
    TP=TIME(I)
    DT=TIME(I)-TIME(I-1)
    IF(TP.LT.T2) FU(I)=0.
    IF(TP.LT.T2) GO TO 117
    IF(TP.LE.T1) FU(I)=FUT1*((TP-T2)/(T1-T2))**2
C INTERPOLATE THE CORRECTION FACTOR OF DECAY HEAT
CALL DATSG
1      (TIME(I),Z,F,WORK,IROW,ICOL,ARG,VAL,NDIM)
CALL DALI(TIME(I),ARG,VAL,Y,NDIM,EPS,IER)
RWR=PWR*Y
IF(TP.GT.T1)FU(I)=FUT1+      ( 9.183E-5*RWR*H/EMU)*(DEXP(.
```

```

1709*DLOG(TP))-DEXP(.709*DLOG(T1)))
1117 IF((XLAMDA*TP).GT.70.) EXL=0.
      IF((XLAMDA*TP).GT.70.) GO TO 117
      IF((XLAMDA*TP).LT.1.E-6) EXL=1.-XLAMDA*TP
      IF((XLAMDA*TP).LT.1.E-6) GO TO 117
      EXL=DEXP(-XLAMDA*TP)
117  IF(TP.LE.T2) Q(I)=QZERO*EXL
      IF(TP.LE.T2) GO TO 80
      X4=-FP*DLOG(1.-FU(I))+XLAMDA*TP
      IF(X4.GT.70.) EX4=0.
      IF(X4.GT.70.) GO TO 119
      IF(X4.LE.1.E-6) EX4=1.-X4
      IF(X4.LE.1.E-6) GO TO 119
      EX4=DEXP(-X4)
119  IF(TP.GT.T2) Q(I)=QZERO*EX4
80   IF(Q(I).GT.QZERO*EXL) Q(I)=QZERO*EXL
      IF(EXL.EQ.0.) FI(I)=1.
      IF(EXL.NE.0.) FI(I)=(QZERO*EXL-Q(I))/(QZERO*EXL)
      YP = ELP/VP
      ZP=ELP/VC
      IF(IM.EQ.4)AXP=YP+XLAMDA+DNP
      IF(IM.NE.4)AXP=YP+XLAMDA
      FMU=(1-FU(I))*EMU
      IF(TP.LE.T1) YQ=ELUT1*FP*(TP-T2-DT/2.)/(FMU*(T1-T2))
      IF(TP.LE.T2) YQ=0.
      IF(YQ.LE.0.) YQ=0.
      ELU=6.511E-5*RWR*H*(1.-(FU(I)+FU(I-1))/2.)*DEXP(-.291*DLOG(TP-DT/2
1.))
      IF(TP.GT.T1) YQ=ELU*FP/FMU
      QR=Q(I)-(Q(I)-Q(I-1))/U
      IF(QR.LT.1.E-30) QR=0.
      IF(I.EQ.2) CR=C(I-1)
      IF(I.NE.2) CR=C(I-1)+(C(I-1)-C(I-2))/U
      IF(CR.LT.1.E-30) CR=0.
      XP=DSORT(4*D*(YQ*QR+ZP*CR)+AXP**2)
      IF((XP*DT).GT.70.) EXP=0.

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      IF((XP*DT).GT.70.) GO TO 120
      IF((XP*DT).LE.1.E-8) EXP=1.-XP*DT
      IF((XP*DT).LE.1.E-6) GO TO 120
      EXP=DEXP(-XP*DT)
120  PE=XP/(2*D)
      PD=-AXP/(2*D)
      PX=P(I-1)-PC
      PP=PX+PE
      PM=PX-PE
125  IF(P(I-1).LE.1.E-30) P(I-1)=0.
      P(I)=PD+PE*(PP+PM*EXP)/(PP-PM*EXP)
      IF (IM.EQ.4) GC TO 106
      IF ((AXP*DT).GT.70.) EXP=0.
      IF ((AXP*DT).GT.70.) EXPP=1.
      IF ((AXP*DT).GT.70.) EXPPP=1./AXP
      IF ((AXP*DT).GT.70.)GC TO 1104
      IF ((AXP*DT).LE.1.E-8) EXP=1.-AXP*DT
      IF ((AXP*DT).LE.1.E-8) EXPP=AXP*DT
      IF ((AXP*DT).LE.1.E-8) EXPPP=DT
      IF ((AXP*DT).LE.1.E-8) GO TO 1104
      EXP=DEXP(-AXP*DT)
      EXPP=1.-EXP
      EXPPP=EXPP/AXP
1104 PD=((YQ*QR)+(ZP*CR))*EXPPP
      P(I)=PD+P(I-1)*EXP
106  IF (IC.GT.2) GO TO 90
      FLTIT=FLTIC
      FLTRT=FLTRC
      VT=VC
      ELT=ELC
      ELB=ELP
      YB=ELP/VP
      ZB=ZP
      B(I)=P(I)
      GO TO 92
90   YC=ELC/VC
      ZC=ELC/VB

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IF (IM.NE.1) AXC=YC+XLAMDA+DNC +ZP+FLTRC*EFF/VC
IF (IM.EQ.1) AXC=YC+XLAMDA+ZP
IF (IM.NE.3) GO TO 95
IF (IM3.EQ.0) AXC=YC+XLAMDA+FLTIC*EFFI/VC+ZP
95 PR=P(I)-(P(I)-P(I-1))/U
IF (PR.LT.1.E-30) PR=0.
IF (I.EQ.2) BR=0.
IF (I.EQ.2) GO TO 129
IF (IC.NE.3) BR=B(I-1)+(B(I-1)-B(I-2))/U
IF (IC.EQ.3) BR=T(I-1)+(T(I-1)-T(I-2))/U
129 IF (BR.LT.1.E-30) BR=0.
XC=DSQRT(4*D*(YP*PR+ZC*BR)+AXC**2)
IF ((XC*DT).GT.70.) EXC=0.
IF ((XC*DT).GT.70.) GO TO 121
IF ((XC*DT).LE.1.E-6) EXC=1.-XC*DT
IF ((XC*DT).LE.1.E-6) GO TO 121
EXC=DEXP(-XC*DT)
121 CE=XC/(2*D)
CD=-AXC/(2*D)
CX=C(I-1)-CD
CP=CX+CE
CM=CX-CE
126 IF (C(I-1).LE.1.E-30) C(I-1)=0.
C(I)=CD+CE*(CP+CM*EXC)/(CP-CM*EXC)
IF (IM.EQ.2) GO TO 1002
IF (IM.EQ.4) GO TO 1002
IF (IM.EQ.1) GO TO 1011
IF (IM3.NE.0) GO TO 1002
1011 IF ((AXC*DT).GT.70.) EXC=0.
IF ((AXC*DT).GT.70.) GO TO 1001
IF ((AXC*DT).LE.1.E-6) EXC=1.-AXC*DT
IF ((AXC*DT).LE.1.E-6) GO TO 1001
EXC=DEXP(-AXC*DT)
1001 CD= ((YP*PR)+ZC*BR) /AXC*(1.-EXC)
C(I)=CD+C(I-1)*EXC
1002 IF (IC.GT.3) GO TO 93

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      B(I)=C(I)
      FLTIT=FLTIB
      FLTRT=FLTRB
      VT=VB
      ELT=ELB
      YB=ELC/VC
      ZB=ZC
      GO TO 92
93  YB=ELB/VB
      ZB=ELB/VT
      IF (IM.NE.1) AXB=YB+XLAMDA+DNB +ZC+FLTRB*EFF/VB
      IF (IM.EQ.1) AXB=YB+XLAMDA+ZC
      IF (IM.NE.3) GO TO 96
      IF (IM3.EQ.0) AXB=YB+XLAMDA+FLTIB*EFFI/VB+ZC
96  CR=C(I)-(C(I)-C(I-1))/U
      IF (CR.LT.1.E-30) CR=0.
      IF (I.EQ.2) TR=T(I-1)
      IF (I.NE.2) TR=T(I-1)+(T(I-1)-T(I-2))/U
      IF (TR.LT.1.E-30) TR=0.
      XB=DSQRT(4*D*(YC*CR+ZB*TR)+AXB**2)
      IF ((XB*DT).GT.70.) EXB=0.
      IF ((XB*DT).GT.70.) GO TO 122
      IF ((XB*DT).LE.1.E-6) EXB=1.-XB*DT
      IF ((XB*DT).LE.1.E-6) GO TO 122
      EXB=DEXP(-XB*DT)
122 BE=XB/(2*D)
      BD=-AXB/(2*D)
      BX=B(I-1)-BD
      BP=BX+BE
      BM=BX-BE
127 IF (B(I-1).LE.1.E-30) B(I-1)=0.
      B(I)=BD+BE*(BP+BM*EXB)/(BP-BM*EXB)
      IF (IM.EQ.2) GO TO 92
      IF (IM.EQ.4) GO TO 92
      IF (IM.EQ.1) GO TO 1013
      IF (IM3.NE.0) GO TO 92
1013 IF ((AXB*DT).GT.70.) EXB=0.
      IF ((AXB*DT).GT.70.) GO TO 1003

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      IF ((AXB*DT).LE.1.E-6) EXB=1.-AXB*DT
      IF ((AXB*DT).LE.1.E-6) GO TO 1003
      EXB=CEXP(-AXB*DT)
1003  BD=((YC*CR)+(ZB*TR))/AXB*(1.-EXB)
      B(I)=BD+B(I-1)*EXB
      92  YT=ELT/VT
      IF (IM.NE.1) AXT=YT+XLAMDA+DNT +ZB+FLTRT*EFF/VT
      IF (IM.EQ.1) AXT=YT+XLAMDA+ZB
      IF (IM.NE.3) GO TO 101
      IF (IM3.EQ.0) AXT=YT+XLAMDA+FLTIT*EFFI/VT+ZB
101  BR=B(I)-(B(I)-B(I-1))/U
      IF (BR.LT.1.E-30) BR=0.
      XT=DSQRT(4*D*YB*BR+AXT**2)
      IF ((XT*DT).GT.70.) EXT=0.
      IF ((XT*DT).GT.70.) GO TO 123
      IF ((XT*DT).LE.1.E-6) EXT=1.-XT*DT
      IF ((XT*DT).LE.1.E-6) GO TO 123
      EXT=CEXP(-XT*DT)
123  TE=XT/(2*D)
      TD=-AXT/(2*D)
      TX=T(I-1)-TD
      TP=TX+TE
      TM=TX-TE
128  IF (T(I-1).LE.1.E-30) T(I-1)=0.
      T(I)=TD+TE*(TP+TM*EXT)/(TP-TM*EXT)
      IF (IM.EQ.2) GC TC 1006
      IF (IM.EQ.4) GO TO 1006
      IF (IM.EQ.1) GC TC 1015
      IF (IM3.NE.0) GO TO 1006
1015 IF ((AXT*DT).GT.70.) EXT=0.
      IF ((AXT*DT).GT.70.) GO TO 1005
      IF ((AXT*DT).LE.1.E-6) EXT=1.-AXT*DT
      IF ((AXT*DT).LE.1.E-6) GO TO 1005
      EXT=CEXP(-AXT*DT)
1005 TB=(YB*BR/AXT)*(1.-EXT)
      T(I)=TB+T(I-1)*EXT

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1006 IF((XLAMDA*CT).GT.70.) EXR=0.
      IF((XLAMDA*CT).GT.70.) GO TO 124
      IF((XLAMDA*DT).LT.1.E-6) EXR=1.-XLAMDA*DT
      IF((XLAMDA*CT).LT.1.E-6) GO TO 124
      EXR=DEXP(-XLAMDA*CT)
124  TR=T(I)-(T(I)-T(I-1))/U
      IF(TR.LT.1.E-30) TR=0.
      IF(XLAMDA.LE.1.E-6) R(I)=YT*TR*DT+R(I-1)*EXR
      IF(XLAMDA.GT.1.E-6) R(I)=(YT*TR/XLAMDA)*(1.-EXR)+R(I-1)*EXR
      IF(R(I).LE.1.E-30)R(I)=0.
      RR(I)=YT*TR
      IF(IC.EQ.2) SUM=P(I)+T(I)+R(I)
      IF(IC.EQ.3) SUM=P(I)+C(I)+T(I)+R(I)
      IF(SUM.LT.1.E-30) SUM=0.
      IF(IC.EQ.4) SUM=P(I)+C(I)+B(I)+T(I)+R(I)
      IF(EXL.EQ.0.) GO TO 4
      IF(SUM.EQ.0.) GO TO 4
      IF(IMM.EQ.1) W(I)=(SUM+Q(I))/(QZERO*EXL)
4    IF(R(I).LT.1.E-30) FB(I)=0.
      IF(R(I).GE.1.E-30) FB(I)=R(I)/QZERO
      IF(TIME(I)/TSTOP.GE.1.)6.3
6    IF(IM.NE.3) GO TO 5
      IF(IM3.EQ.1) GO TO 305
      DO 54 J=1,I
      CSAV(J) = G * C(J)
      BSAV(J) = G * B(J)
      TSAV(J)=G*T(J)
      RSAV(J) = G * R(J)
      PRSAV(J)=G*RR(J)
54  FBSAV(J)=G*FB(J)
      IM3=1
      GO TO 3000
305 DO 55 J=1,I
      C(J) =(1.-G) * C(J)
      B(J) = (1.-G) * B(J)

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```

T(J)=(1.-G)*T(J)
R(J) = (1.-G) * R(J)
FB(J)=(1.-G)*FB(J)
RR(J)=(1.-G)*RR(J)
C(J) = C(J) + CSAV(J)
B(J) = B(J) + BSAV(J)
T(J)=T(J)+TSAV(J)
R(J) = R(J) + RSAV(J)
RR(J)=RR(J)+RRSAV(J)
55 FB(J)=FB(J)+FBSAV(J)
5 IF( IMM.EQ.1) GO TO 152
IF( IC.NE.3) GO TO 102
DO 103 K=1,I
103 B(K)=0.
102 IF( IC.NE.2) GO TO 105
DO 104 K=1,I
104 C(K)=0.
105 TIME(I+1)=TIME(I)
FU(I+1)=FU(I)
FI(I+1)=FI(I)
FB(I+1)=FB(I)
Q(I+1)=Q(I)
P(I+1)=P(I)
C(I+1)=C(I)
B(I+1)=B(I)
T(I+1)=T(I)
W(I+1)=W(I)
R(I+1)=R(I)
RR(I+1)=R(I)
N2=XN2/10.
N3=XN3/10.
N4=XN4/10.
N5=XN5/10.
N6=XN6/10.

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MM=I+2
MMM=I+42
DO 700 M=MM,MMM
IF(M.LE.(I+12)) L=(M-I-2)*N2+2
IF(M.GT.(I+12)) L=(M-I-2)*N3+2-500
IF(M.GT.(I+22)) L=(M-I-2)*N4+2-500
IF(M.GT.(I+32)) L=(M-I-2)*N5+2+2500
TIME(M)=TIME(L)
FU(M)=FU(L)
FI(M)=FI(L)
FB(M)=FB(L)
Q(M)=Q(L)
P(M)=P(L)
C(M)=C(L)
B(M)=B(L)
T(M)=T(L)
W(M)=W(L)
RR(M)=RR(L)
700 R(M)=R(L)
PRINT 59,(TITLE(K),K=1,10),III
59 FORMAT (1H1,5X,10A8/6X,5HCASE ,I2)
PRINT 56,XLAMDA,QZERO,FP,G,IM
56 FORMAT (1H0,5X,'DECAY CONSTANT, 1/SEC',29X,E12.3/6X,'CORE INVENTOR
1Y AT ZERO TIME, CURIES',15X,E12.3/6X,'RATIO OF FP VAPOR PRESSURE T
20 UO2 VAPOR PRESSURE',2X,E12.3/6X,'FRACTION OF FP WHICH IS GASEOUS
3',19X,E12.3/6X,'CLASS OF FISSION PRODUCT',36X,I2)
PRINT 24,(TIME(K),FU(K),FI(K),
1 FB(K),Q(K),P(K),C(K),B(K),T(K),R(K),R
2R(K),W(K),K=MM-1,MMM)
24 FORMAT (1H0,6X,'TIME',6X,'FRACTION',2X,'FRACTION',2X,'FRACTION',2X
1,'INVENTORIES (CURIES)',30X,'RELEASE'/17X,'OF CORE',3X,'OF FP',5X,
2'OF FP'/7X,'(SEC)',5X,'VAPORIZED',1X,'VAPORIZED',1X,'RELEASED',2X,
3'IN CORE',3X,'IN PV',5X,'IN (2)',4X,'IN (3)',4X,'IN (4)',4X,'(CURI
4ES)',2X,'(CI/SEC)'/((5X,12E10.2))
IF(I.PLOT.EQ.0) GO TO 600
CALL LOGLOG (15,9,5,4,2.667,1,AY)

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```
CALL LETTER (0,80,TITLE(1),AY)
CALL LETTER (1,41,41HTIME AFTER CESSATION OF FISSIONING, (SEC),AY)
CALL LETTER (2,33,33HINVENTORIES AND RELEASE, (CURIES),AY)
DO 60 J=1,I
60 CALL CURVE (J,TIME(J),Q(J),AY)
DO 61 J=1,I
61 CALL CURVE (J,TIME(J),P(J),AY)
DO 62 J=1,I
62 CALL CURVE (J,TIME(J),C(J),AY)
DO 63 J=1,I
63 CALL CURVE (J,TIME(J),B(J),AY)
DO 65 J=1,I
65 CALL CURVE(J,TIME(J),T(J), AY)
DO 64 J=1,I
64 CALL CURVE (J,TIME(J),R(J),AY)
CALL ADVANCE(AY)
600 CONTINUE
500 CONTINUE
GO TO 200
17 END
```

C	SUBROUTINE DATSG(X,Z,F,WORK,IROW,ICCL,ARG,VAL,NDIM)	DTS
C		DTS
C		DTS
	DIMENSION Z(1),F(1),WCRK(1),ARG(1),VAL(1)	DTS
	DOUBLE PRECISION X,Z,F,WORK,ARG,VAL,B,DELTA	DTS
	IF(IROW)11,11,1	DTS
	1 N=NDIM	DTS
C	IF N IS GREATER THAN IROW, N IS SET EQUAL TO IROW.	DTS
	IF(N-IROW)3,3,2	DTS
	2 N=ICW	DTS
C		DTS
C	GENERATION OF VECTOR WORK AND COMPUTATION OF ITS GREATEST ELEMENT.	DTS
	3 B=0.00	DTS
	DO 5 I=1,IROW	DTS
	DELTA=DABS(Z(I)-X)	DTS
	IF(DELTA-B)5,5,4	DTS
	4 B=DELTA	DTS
	5 WORK(I)=DELTA	DTS
C		DTS
C	GENERATION OF TABLE (ARG,VAL)	DTS
	B=B+1.00	DTS
	DO 10 J=1,N	DTS
	DELTA=B	DTS
	DO 7 I=1,IROW	DTS
	IF(WCRK(I)-DELTA)6,7,7	DTS
	6 II=I	DTS
	DELTA=WORK(I)	DTS
	7 CONTINUE	DTS
	ARG(J)=Z(II)	DTS
	IF(ICOL-1)8,9,8	DTS
	8 VAL(2*J-1)=F(II)	DTS
	III=II+IROW	DTS
	VAL(2*J)=F(III)	DTS
	GOTO 10	DTS
	9 VAL(J)=F(II)	DTS
	10 WORK(II)=B	DTS
	11 RETURN	DTS
	END	DTS

C	SUBRCUTINE DALI(X,ARG,VAL,Y,NDIM,EPS,IER)	DAL
C		DAL
C		DAL
	DIMENSION ARG(1),VAL(1)	DAL
	DOUBLE PRECISION ARG,VAL,X,Y,H	DAL
	IER=2	DAL
	DELT2=0.	DAL
	IF(NDIM-1)9,7,1	DAL
C		DAL
C	START OF AITKEN-LOOP	DAL
	1 DO 6 J=2,NDIM	DAL
	DELT1=DELT2	DAL
	IEND=J-1	DAL
	DO 2 I=1,IEND	DAL
	H=ARG(I)-ARG(J)	DAL
	IF(H)2,13,2	DAL
	2 VAL(J)=(VAL(I)*(X-ARG(J))-VAL(J)*(X-ARG(I)))/H	DAL
	DELT2=DABS(VAL(J)-VAL(IEND))	DAL
	IF(J-2)6,6,3	DAL
	3 IF(DELT2-EPS)10,10,4	DAL
	4 IF(J-8)6,5,5	DAL
	5 IF(DELT2-DELT1)6,11,11	DAL
	6 CONTINUE	DAL
C	END OF AITKEN-LOOP	DAL

C	7 J=NDIM	DAL
	8 Y=VAL(J)	DAL
	9 RETURN	DAL
C		DAL
C	THERE IS SUFFICIENT ACCURACY WITHIN NDIM-1 ITERATION STEPS	DAL
10	IER=0	DAL
	GOTO 8	DAL
C		DAL
C	TEST VALUE DELT2 STARTS OSCILLATING	DAL
11	IER=1	DAL
12	J=IEND	DAL
	GOTO 8	DAL
C		DAL
C	THERE ARE TWO IDENTICAL ARGUMENT VALUES IN VECTOR ARG	DAL
13	IER=3	DAL
	GOTO 12	DAL
	END	DAL

```

**FTN,L,E,M,G.
PROGRAM FEED
IMPLICIT REAL * 8(A-H,C-Z)
DIMENSION TIME(4550),FU(4550),FI(4550),Q(4550),Z(34),F(34),
IWORK(34),ARG(5),VAL(5),DMULT(4550),RDMULT(4550)
TSTART=10.
XN2=1000.
XN3=1500.
XN4=1500.
XN5=500.
XN6=20.
T2=10.
T1=60.
TSTOP=.999E5
EMU = 1.48 E8
PWR=3.44E9
H=1.E-3
INDI=1
READ 2,ICOL,IRCW,NDIM,EPS,ISET
2 FORMAT(3I10,E10.2,I10)
DO 100 I=1,4550
100 RDMULT(I)=1.
110 DO 120 I=1,4550
120 DMULT(I)=0.
5 READ(50,58,END=17) IM,XLAMDA,QZERO,FP
58 FORMAT (I10,3E10.2)
READ 4,Z
4 FORMAT (8E10.4)
READ 6,F
6 FORMAT (8E10.4)
DO 10 I=1,4550
FI(I)=0.
10 TIME(I)=0.
ELUTI=6.511E-5*PWR*DEXP(-.291*DLOG(T1))*H*RDMULT(780)
201 ELUTI=ELUTI*EXPF(-25.*ELUTI/EMU)
FUT1=ELUTI*(T1-T2)/(2.*EMU)

```

```

3000 I = 1
      3 I = I + 1
      XI=I-2
      IF(TIME(I-1).LT..999E2) TIME(I)=TSTART*(10.**(XI/XN2))
      IF(TIME(I-1).GE..999E2) TIME(I)=TIME(I-1)*(10.**(1./XN3))
      IF(TIME(I-1).GE..999E3) TIME(I)=TIME(I-1)*(10.**(1./XN4))
      IF(TIME(I-1).GE..999E4) TIME(I)=TIME(I-1)*(10.**(1./XN5))
      IF(TIME(I-1).GE..999E5) TIME(I)=TIME(I-1)*(10.**(1./XN6))
      TP=TIME(I)
      DT=TIME(I)-TIME(I-1)
      RWR=PWR*RDMULT(I)
      IF(TP.LT.T2) FU(I)=0.
      IF(TP.LT.T2) GO TO 117
      IF(TP.LE.T1) FU(I)=FUT1*((TP-T2)/(T1-T2))**2
      IF(TP.GT.T1) FU(I)=1.-(1.-FUT1)*DEXP((-9.183E-5*RWR*H/EMU)*(DEXP(.
1709*DLOG(TP))-DEXP(.709*DLOG(T1))))
1117 IF((XLAMDA*TP).GT.70.) EXL=0.
      IF((XLAMDA*TP).GT.70.) GO TO 117
      IF((XLAMDA*TP).LT.1.E-6) EXL=1.-XLAMDA*TP
      IF((XLAMDA*TP).LT.1.E-6) GO TO 117
      EXL=DEXP(-XLAMDA*TP)

117 IF(TP.LE.T2) Q(I)=QZERO*EXL
      IF(TP.LE.T2) GO TO 80
      X4=-FP*DLOG(1.-FU(I))+XLAMDA*TP
      IF(X4.GT.70.) EX4=0.
      IF(X4.GT.70.) GO TO 119
      IF(X4.LE.1.E-6) EX4=1.-X4
      IF(X4.LE.1.E-6) GO TO 119
      EX4=DEXP(-X4)

119 IF(TP.GT.T2) Q(I)=QZERO*EX4
      80 IF(Q(I).GT.QZERO*EXL) Q(I)=QZERO*EXL
      IF(EXL.EQ.0.)FI(I)=1.
      IF(EXL.NE.0.)FI(I)=(QZERO*EXL-Q(I))/(QZERO*EXL)
      CALL DATSG(TIME(I),Z,F,WORK,IROW,ICCL,ARG,VAL,NDIM)
      CALL DALI(TIME(I),ARG,VAL,Y,NDIM,EPS,IER)

```

```

DMULT(I)=(1.-FI(I))*Y+DMULT(I)
IF (TIME(I).LT.TSTCP) GO TO 3
ISAV=I
TIME(I+1)=TIME(I)
FI(I+1)=FI(I)
DMULT(I+1)=DMULT(I)
N2=XN2/10.
N3=XN3/10.
N4=XN4/10.
N5=XN5/10.
N6=XN6/10.
MM=I+2
MMM=I+42
DO 700 M=MM,MMM
IF (M.LE.(I+12)) L=(M-I-2)*N2+2
IF (M.GT.(I+12)) L=(M-I-2)*N3+2-500
IF (M.GT.(I+22)) L=(M-I-2)*N4+2-500
IF (M.GT.(I+32)) L=(M-I-2)*N5+2+2500
TIME(M)=TIME(L)
700 FI(M)=FI(L)
PRINT 20,INDI,IM
20 FORMAT(1H1,4X,'INDI = ',I10/5X,'IM = ',I10)
PRINT 30,(TIME(K),FI(K),K=MM-1,MMM)
30 FORMAT(1H0,9X,'TIME',6X,'FRACTION'/20X,'OF FP'/7X,'(SEC)',8X,
1'VAPORIZED'/(5X,E10.4,8X,E10.4))
IF (IV.LT.10)GO TO 5
DO 15 I=1,4550
15 RDMULT(I)=DMULT(I)
I=ISAV
DO 800 M=MM,MMM
IF (M.LE.(I+12)) L=(M-I-2)*N2+2
IF (M.GT.(I+12)) L=(M-I-2)*N3+2-500
IF (M.GT.(I+22)) L=(M-I-2)*N4+2-500
IF (M.GT.(I+32)) L=(M-I-2)*N5+2+2500
TIME(M)=TIME(L)

```

```
800 RDMULT(M)=RDMULT(L)
    PRINT 40 ,INDI
40  FORMAT(1H1,5X,'INDI = ',I10)
    PRINT 50,(TIME(K),RDMULT(K),K=MM-1,MMM)
50  FORMAT(1H0,6X,'TIME',9X,'MULTIPLIER'/(5X,E10.4,8X,E10.4))
    IF(INDI.LT.ISET) INDI=INDI+1
    IF(INDI.LT.ISET) GO TO 110
17  END
```


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